

SNUPPS

Standardized Nuclear Unit
Power Plant System

5 Choke Cherry Road
Rockville, Maryland 20850
(301) 869-8010

April 6, 1984

Nicholas A. Petrick
Executive Director

SLNRC
SUBJ:

84-0060 FILE: 0541
Chapter 15 Analyses Affected
By Assumption of Zero ppm
Boron Downstream of RWST

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

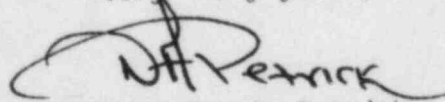
Docket Nos. STN 50-482 and STN 50-483

Reference: SLNRC 84-0042, dated March 14, 1984, Revision Fourteen to
SNUPPS FSAR

Dear Mr. Denton:

The reference transmitted seventy five copies of Revision 14 to the Standardized Nuclear Unit Power Plant System (SNUPPS) FSAR. The revision included an update to the main steam line break (MSLB) analyses to delete credit which had been taken for 2000 ppm Boron in the Boron Injection Tank and the lines downstream of the refueling water storage tank (RWST). The revised figures to accompany the text update missed the printing deadline and were not transmitted. Revised figures for the inadvertent operation of ECCS analysis impact are also provided though the impact is minor and no text changes were required. Copies of the revised figures are enclosed for your information. They will be incorporated in the FSAR via Revision 15, currently scheduled for May, 1984.

Very truly yours,


Nicholas A. Petrick

SLA/n1d2a13
Enclosure: Figures

cc: G. L. Koester
D. T. McPhee
D. F. Schnell

KGE
KCPL
UE

J. Weiser/B. Little
W. Schum/A. Smith
J. Konklin

USNRC/CAL
USNRC/WC
USNRC/RIII

8404100374 840406
PDR ADOCK 05000482
A PDR

Boo!
||

Enclosure

The following revised FSAR Figures are attached:

Figure 15.1-12

Figure 15.1-13

Figure 15.1-15

Figure 15.1-16

Figure 15.1-17

Figure 15.1-18

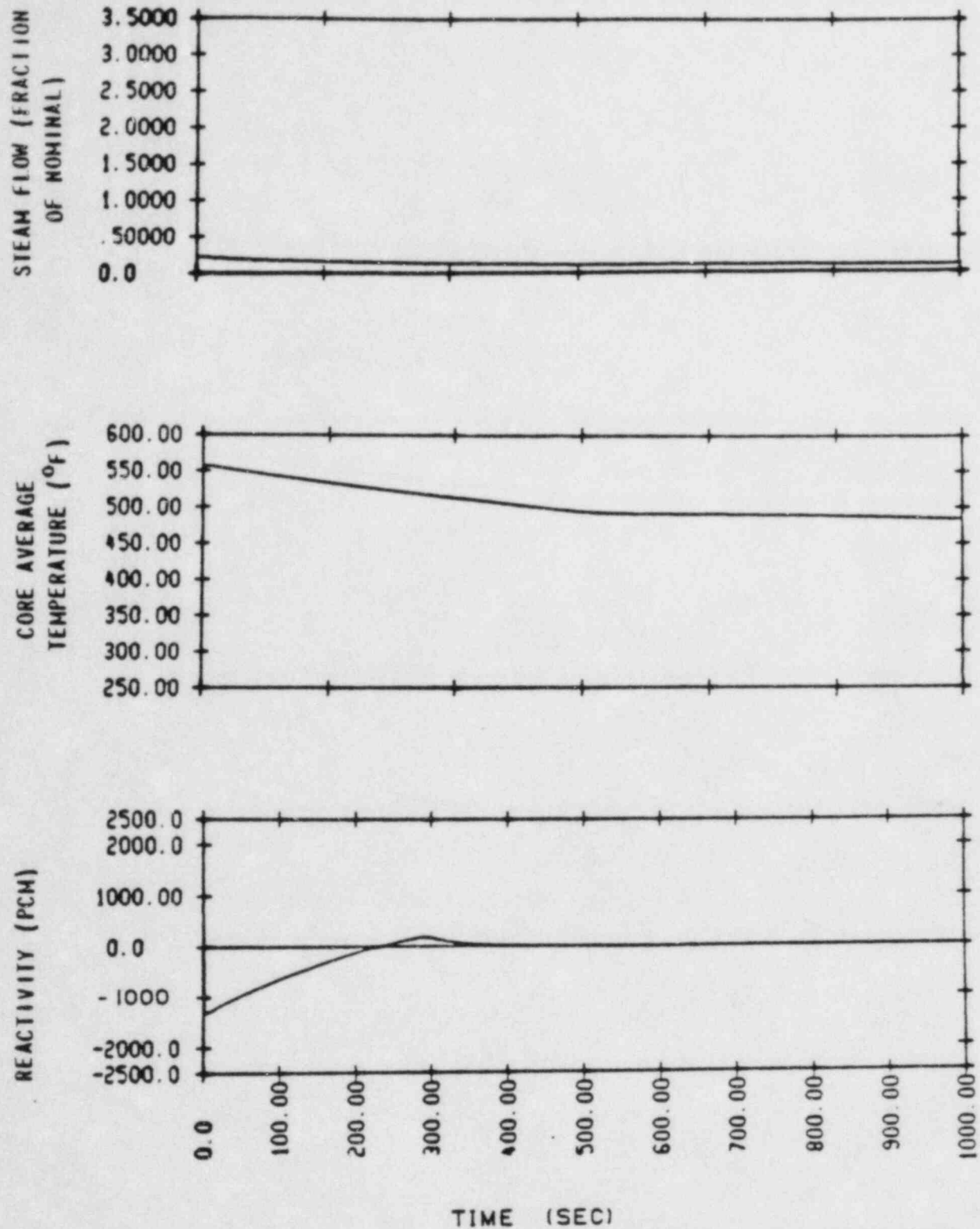
Figure 15.1-19

Figure 15.1-20

Figure 15.5-1

Figure 15.5-2

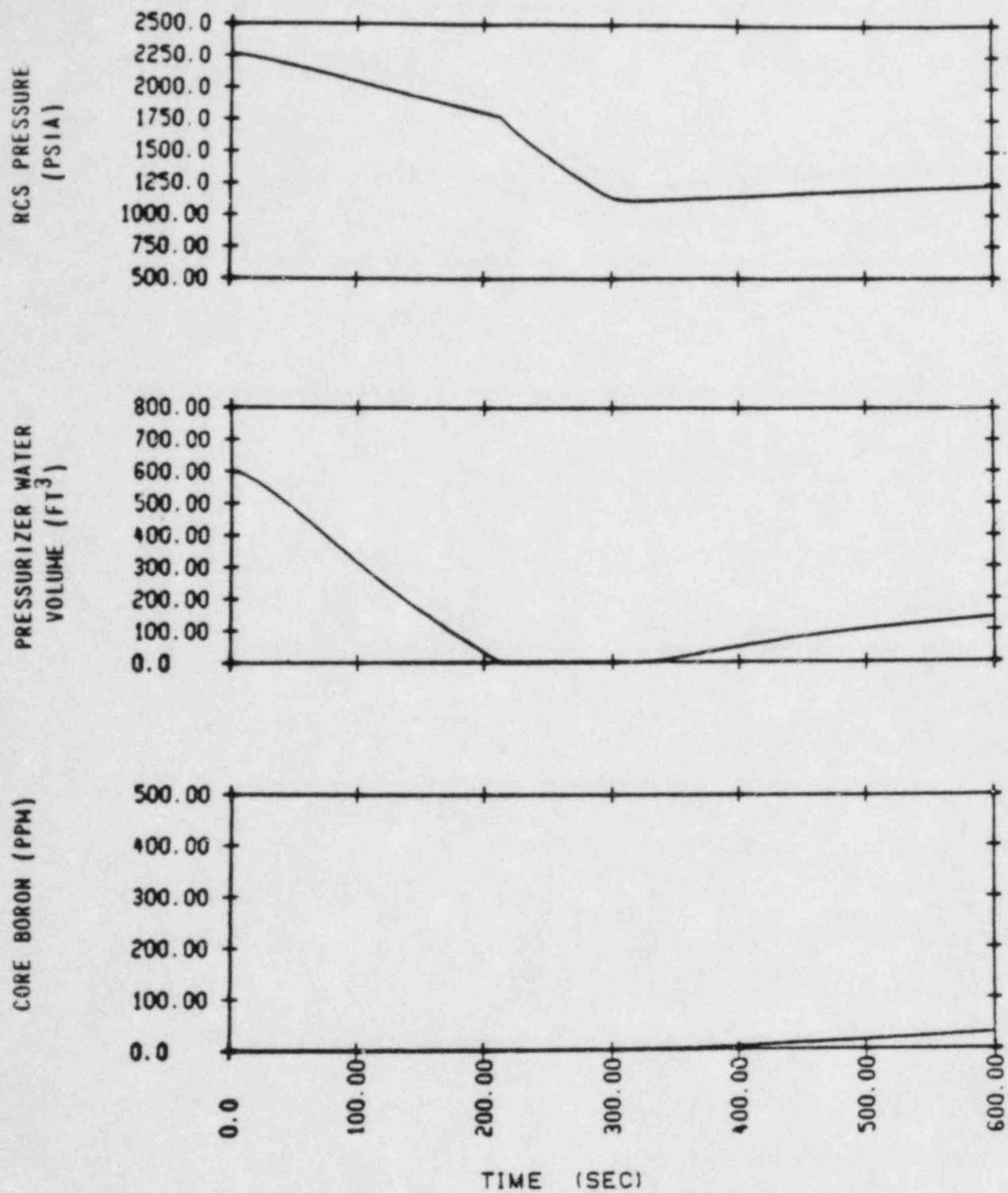
Figure 15.5-3



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FIGURE 15.1-12

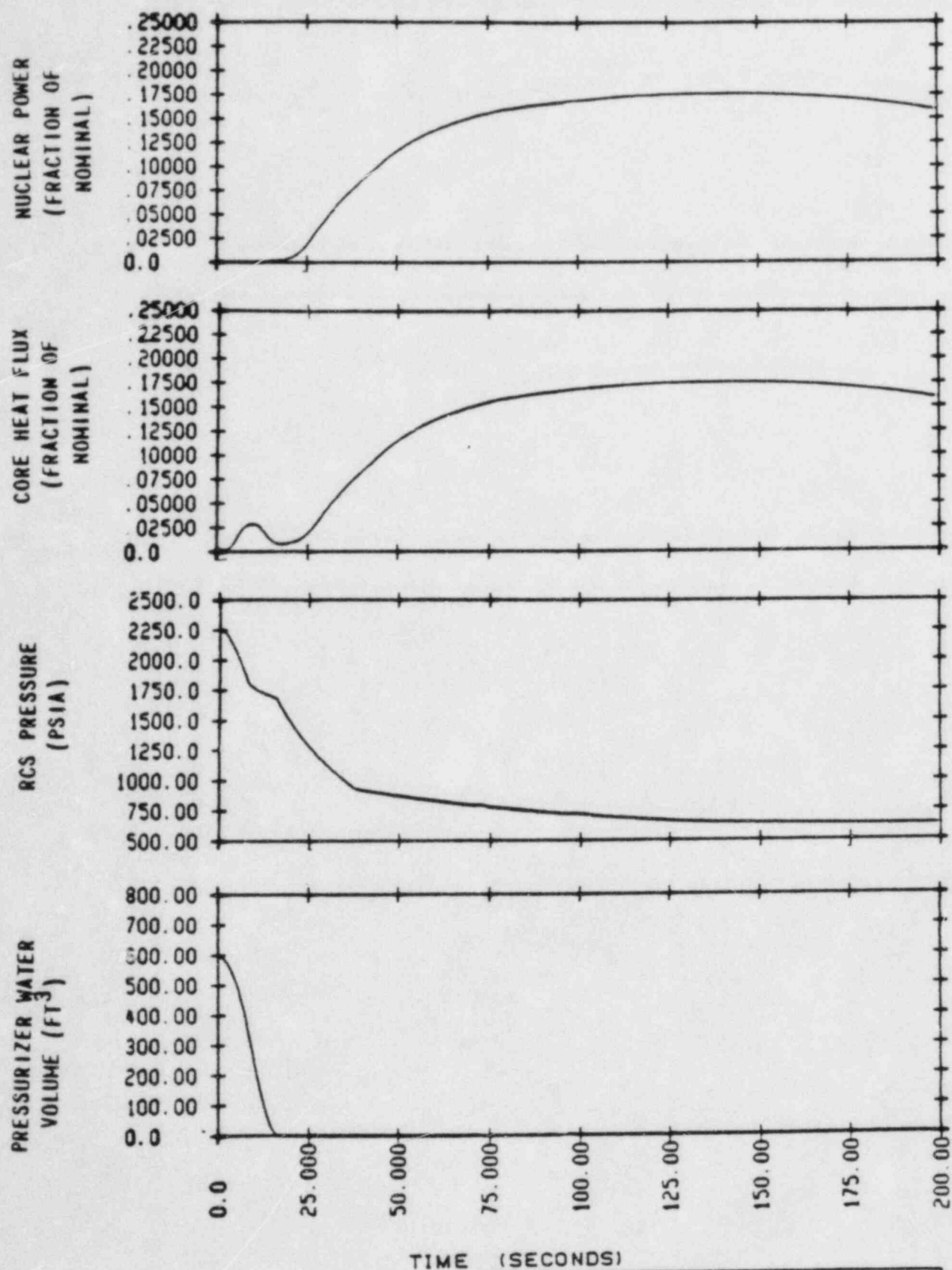
FAILURE OF A STEAM GENERATOR
SAFETY OR RELIEF VALVE



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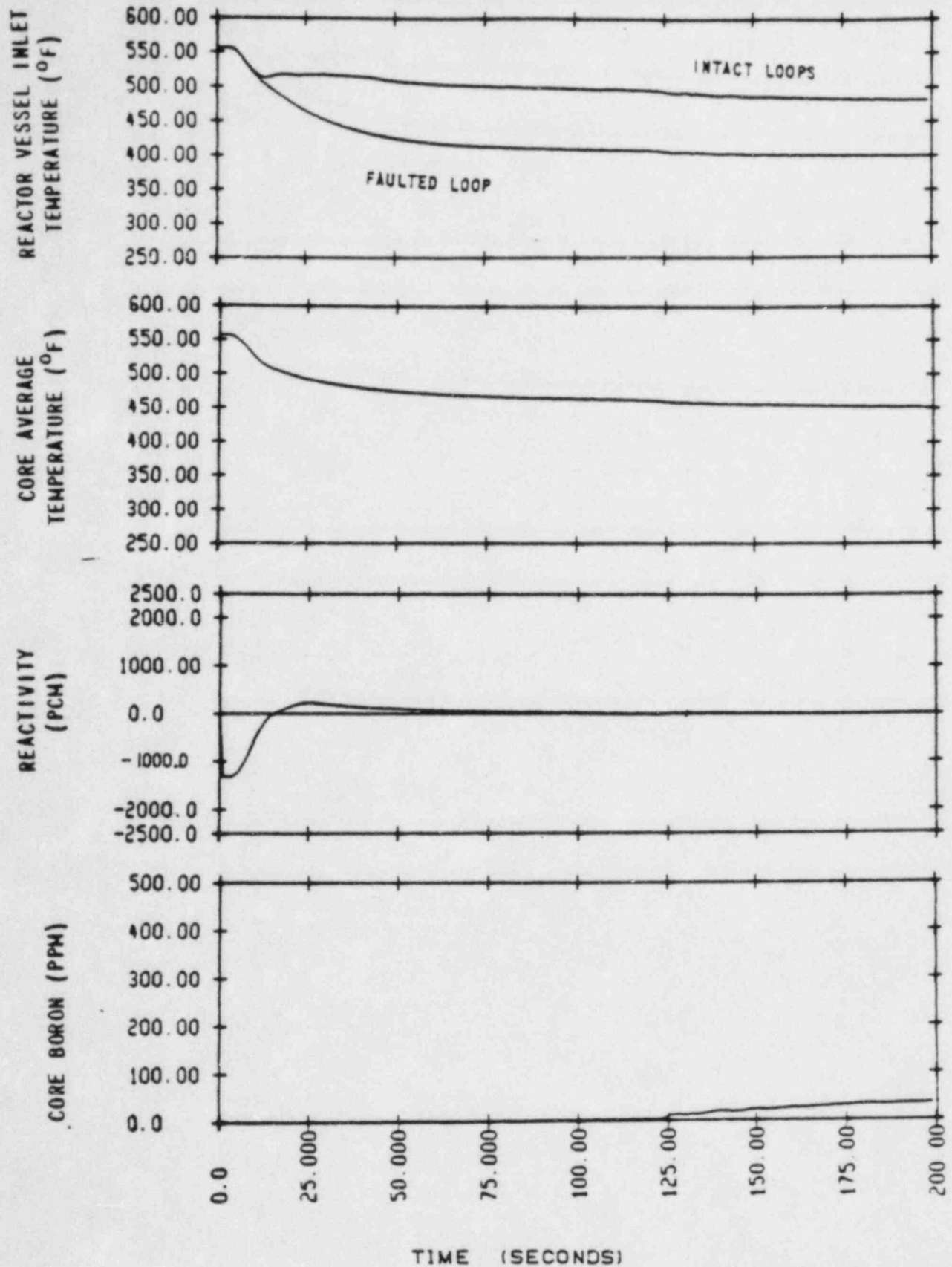
FIGURE 15.1-13

FAILURE OF A STEAM GENERATOR
SAFETY OR RELIEF VALVE



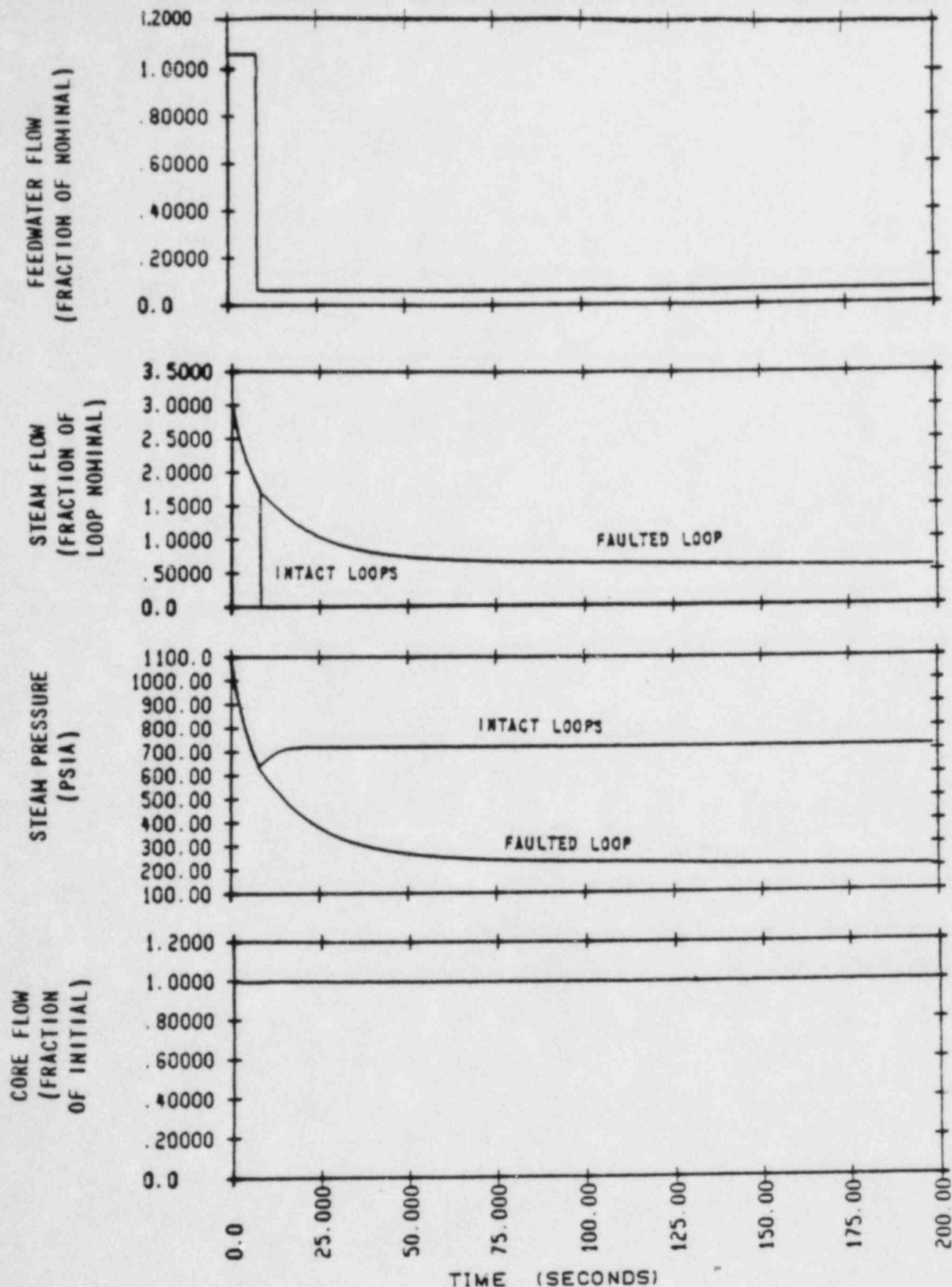
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FIGURE 15.1-15
 NUCLEAR POWER, CORE HEAT FLUX,
 RCS PRESSURE, AND PRESSURIZER WATER
 VOLUME TRANSIENTS FOR 1.4 FT² STEAM
 LINE RUPTURE, OFFSITE POWER AVAILABLE



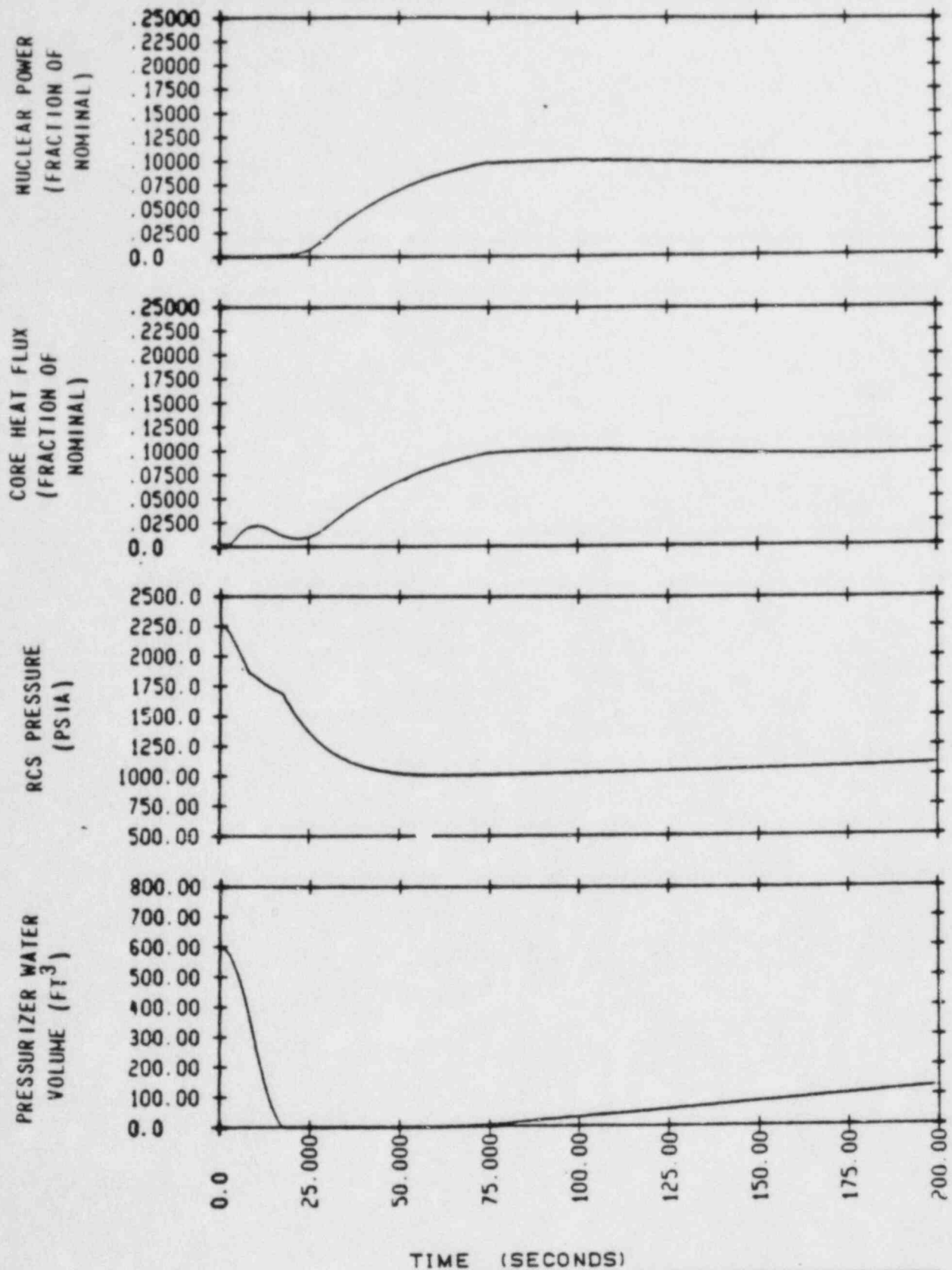
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FIGURE 15.1-16
 REACTOR VESSEL INLET TEMPERATURE,
 CORE AVERAGE TEMPERATURE, REACTIVITY,
 AND CORE BORON TRANSIENTS FOR
 1.4 FT² STEAM LINE RUPTURE,
 OFFSITE POWER AVAILABLE



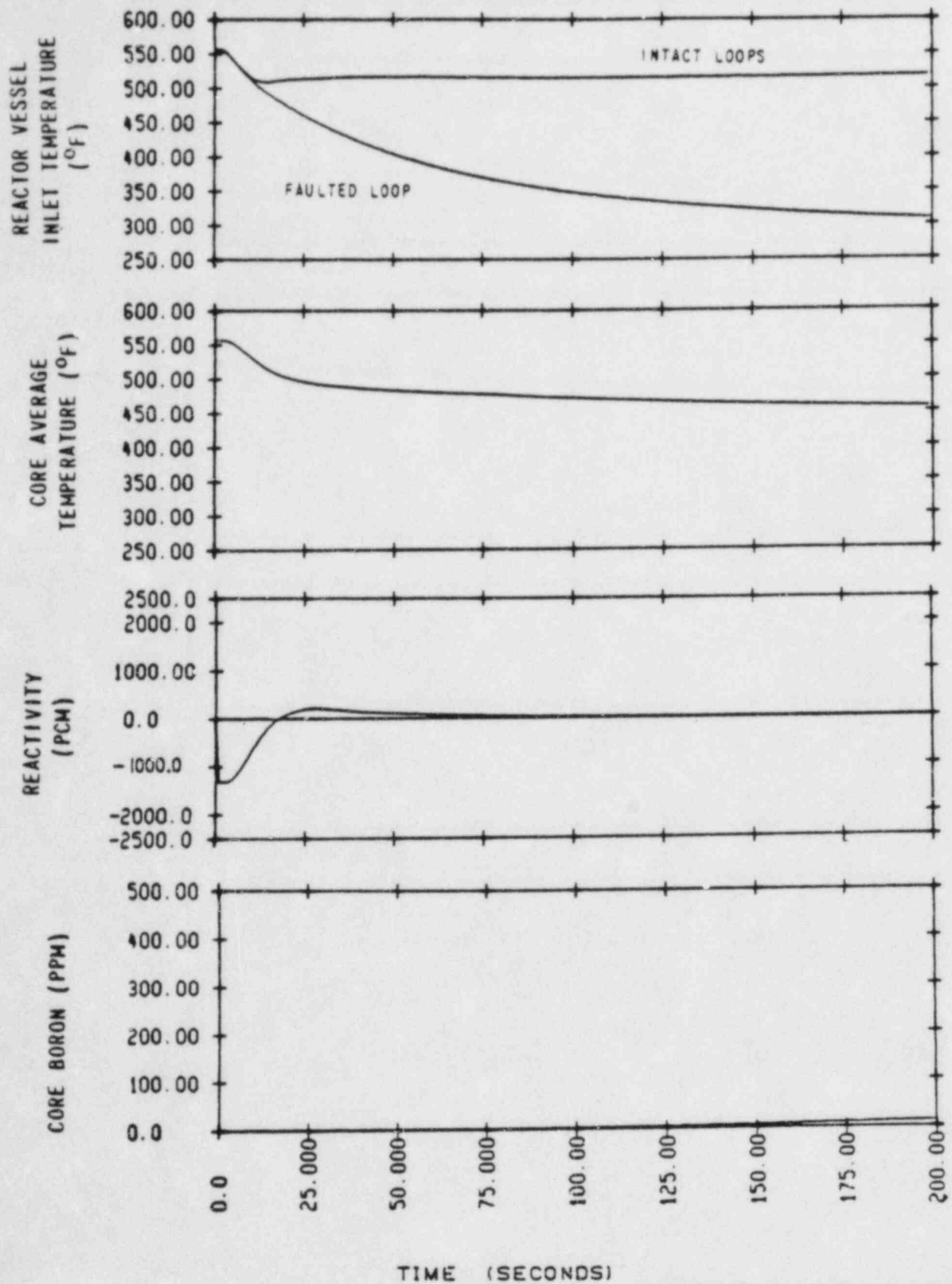
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FIGURE 15.1-17
FEEDWATER FLOW, STEAM FLOW,
STEAM PRESSURE, AND CORE FLOW
TRANSIENTS FOR 1.4 FT² STEAM LINE
RUPTURE, OFFSITE POWER AVAILABLE



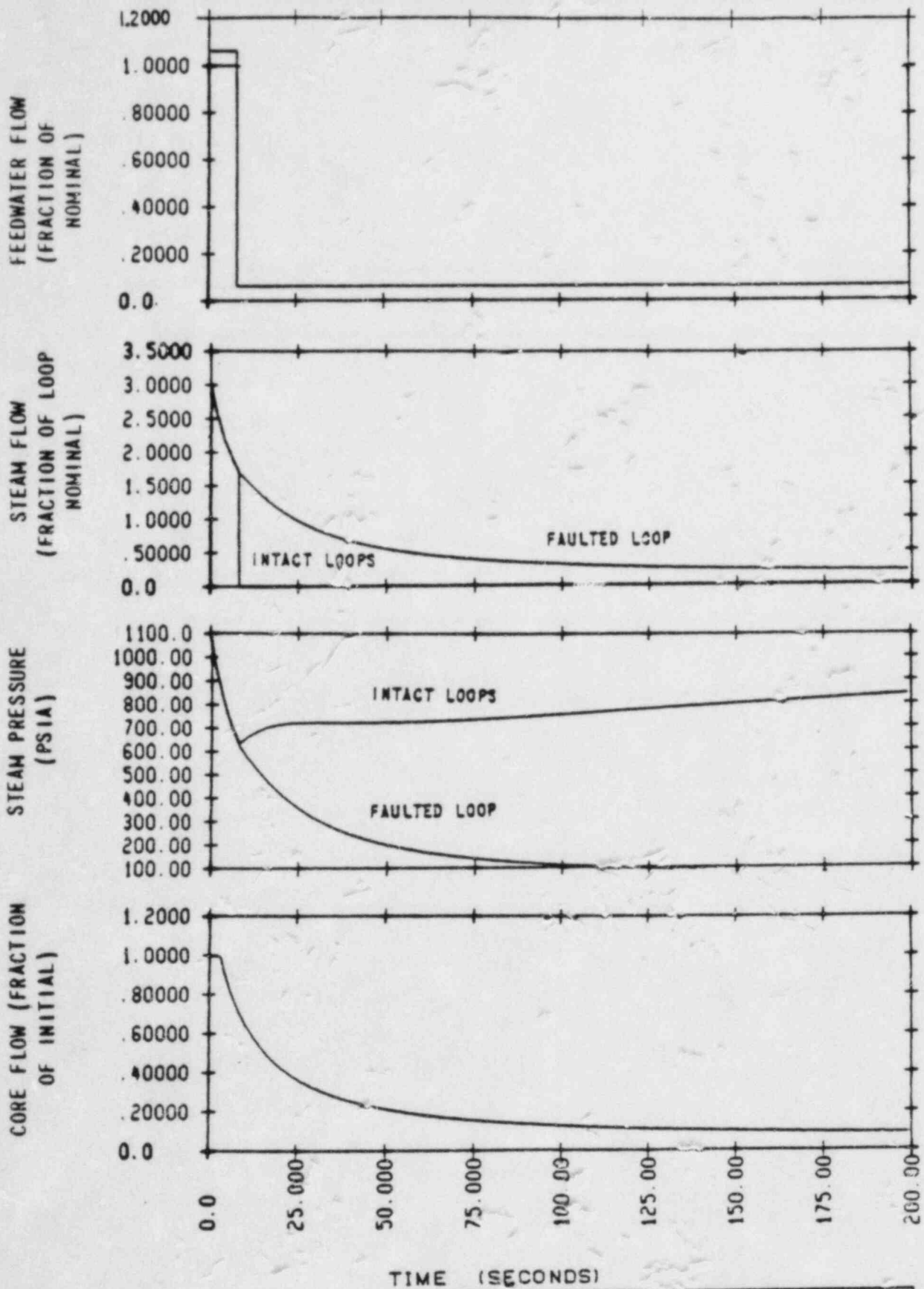
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FIGURE 15.1-18
NUCLEAR POWER, CORE HEAT FLUX,
RCS PRESSURE, AND PRESSURIZER
WATER VOLUME TRANSIENTS FOR 1.4 FT²
STEAM LINE RUPTURE, OFFSITE
POWER NOT AVAILABLE



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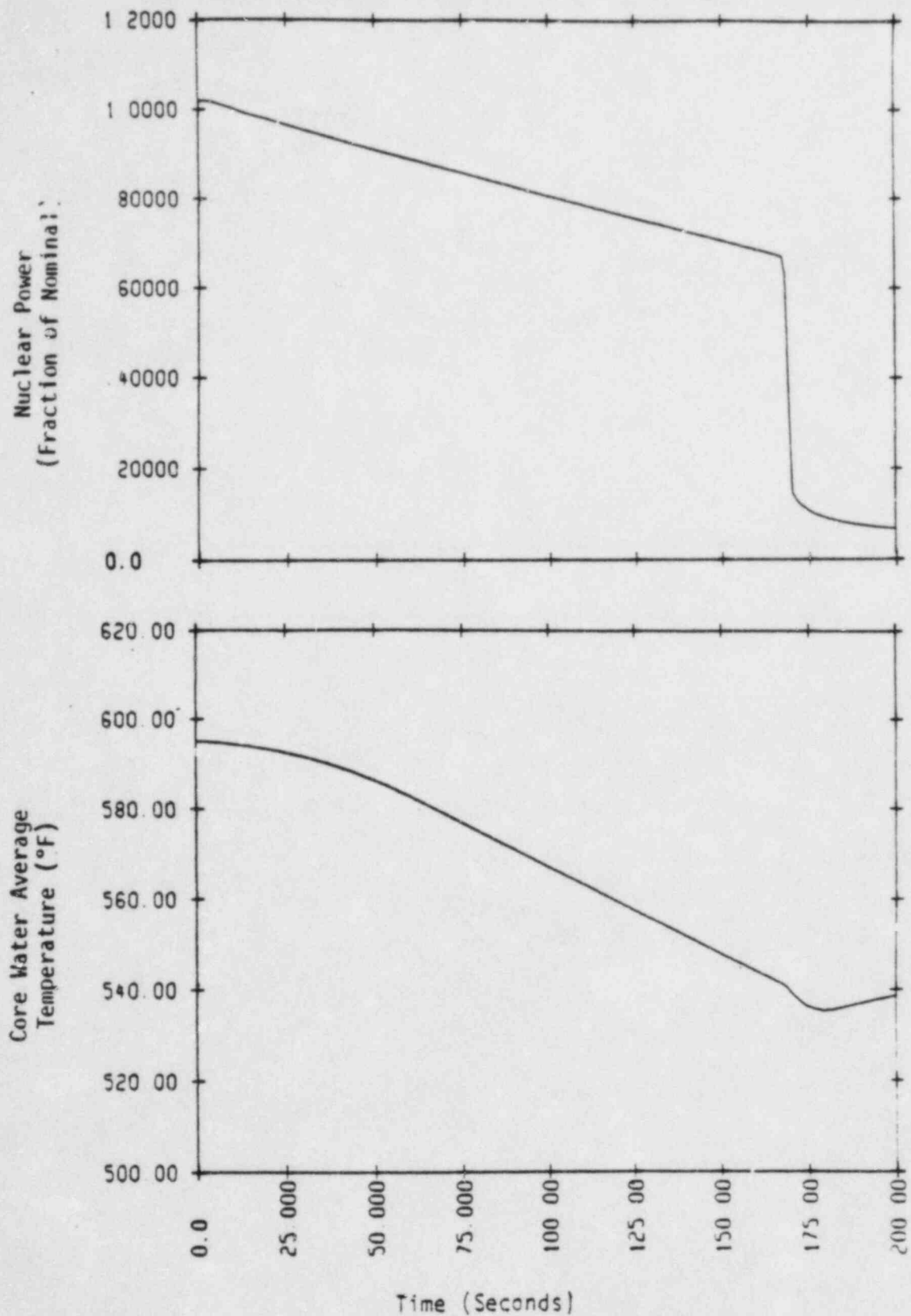
FIGURE 15.1-19
 REACTOR VESSEL INLET TEMPERATURE,
 CORE AVERAGE TEMPERATURE,
 REACTIVITY, AND CORE BORON
 TRANSIENTS FOR 1.4 FT² STEAM LINE
 RUPTURE, OFFSITE POWER NOT AVAILABLE



TIME (SECONDS)

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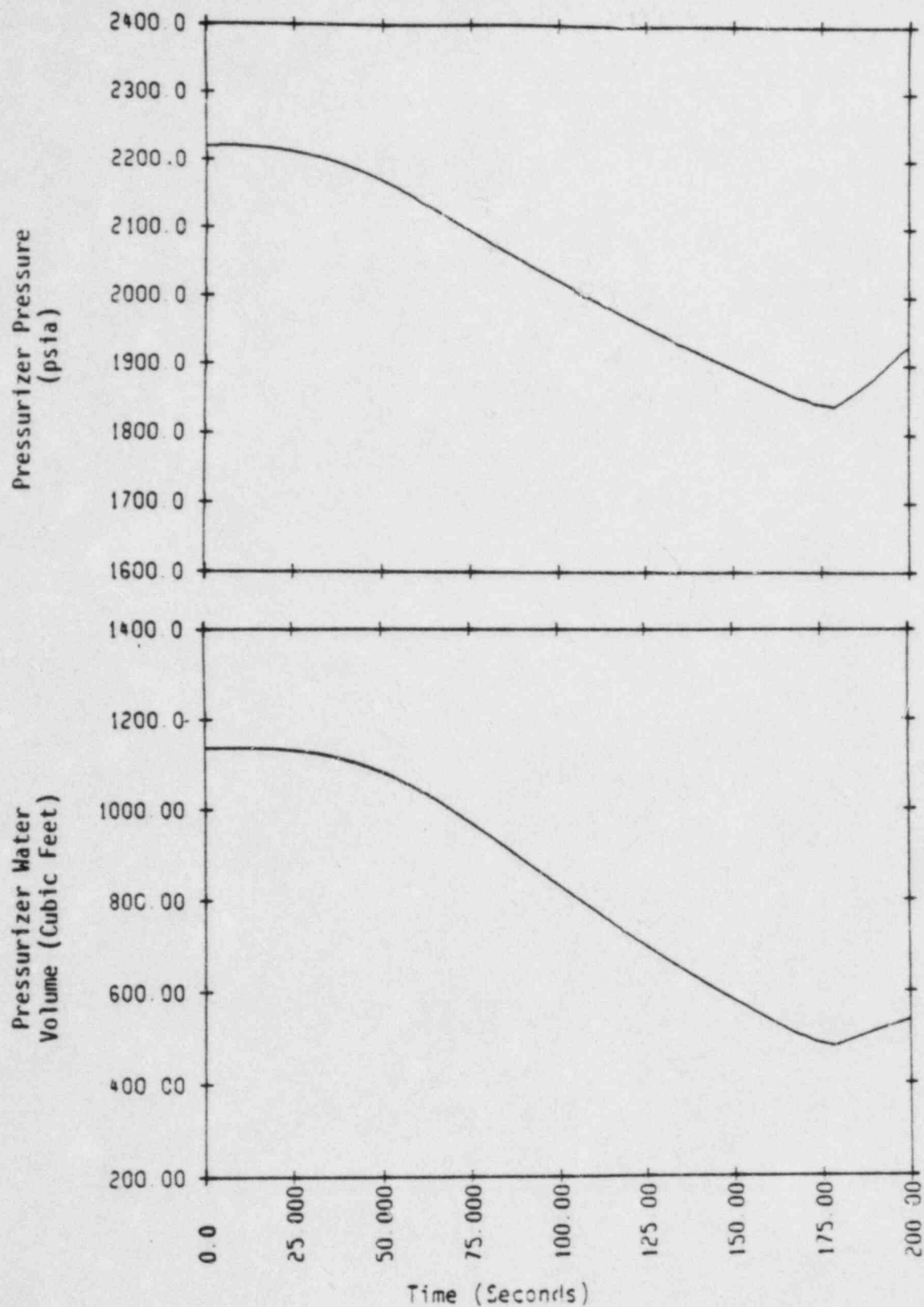
FIGURE 15.1-20
FEEDWATER FLOW, STEAM FLOW,
STEAM PRESSURE, AND CORE FLOW
TRANSIENTS FOR 1.4 FT² STEAM
LINE RUPTURE, OFFSITE POWER
NOT AVAILABLE



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FIGURE 15.5-1

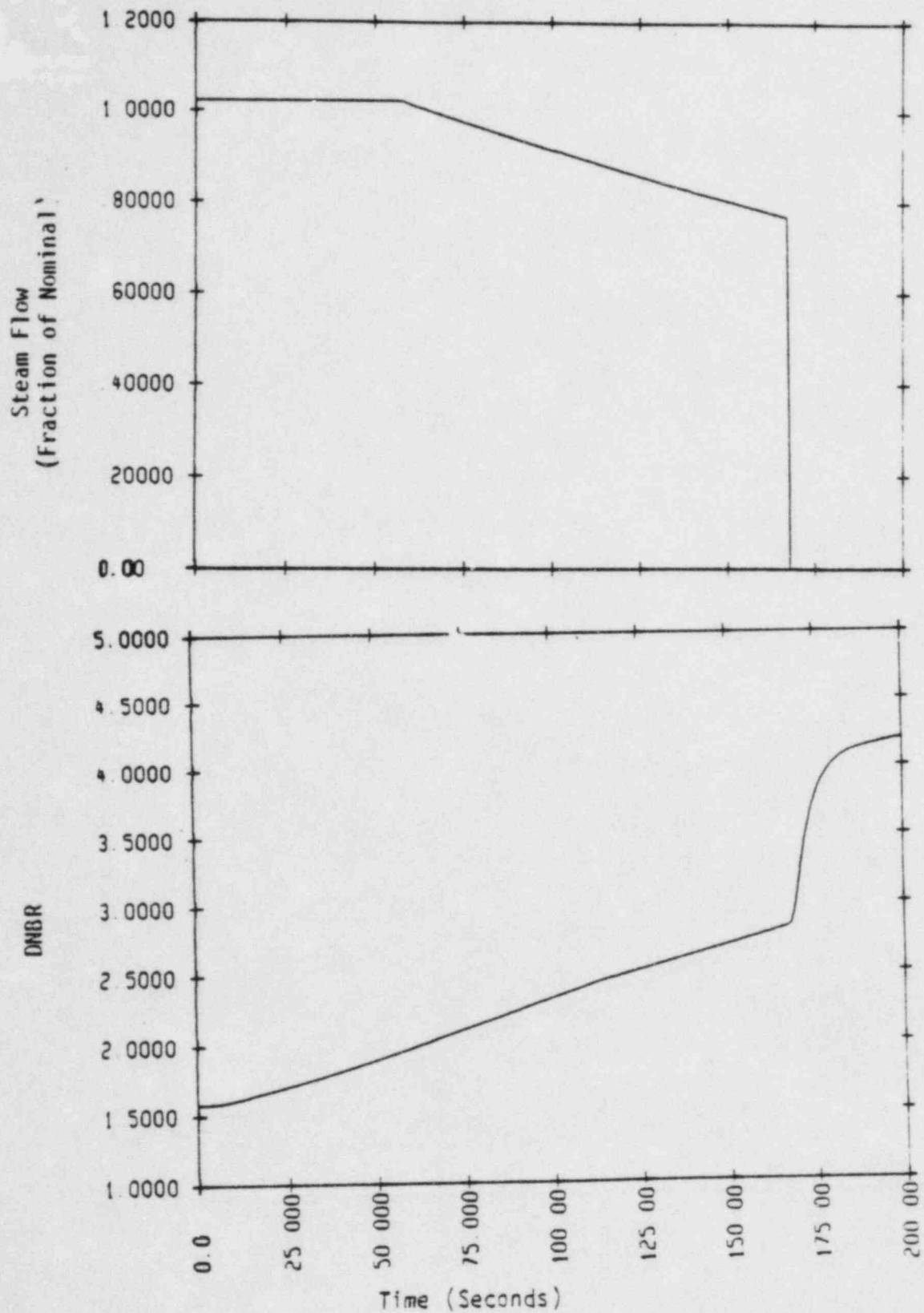
INADVERTENT OPERATION OF ECCS
DURING POWER OPERATION



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FIGURE 15.5-2

INADVERTENT OPERATION OF ECCS
DURING POWER OPERATION



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FIGURE 15.5-3

INADVERTENT OPERATION OF ECCS
DURING POWER OPERATION