

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
DONALD C. COOK NUCLEAR PLANT UNIT-2

DOCKET NUMBER (2)

0 5 0 0 0 3 1 1 6

PAGE (3)

1 OF 0 1

TITLE (4)

ACTUATION OF AN ENGINEERED SAFETY FEATURE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)							
0	3	1	1	8	4	8	4	0	0	3	0	5	0	0	0		
0	3	1	1	8	4	8	4	0	0	3	0	5	0	0	0		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
5			20.402(b)			20.406(c)			<input checked="" type="checkbox"/> 50.73(a)(?) (iv)			73.71(b)					
POWER LEVEL (10)			20.406(a)(1)(i)			50.36(e)(1)			50.73(a)(2)(v)			73.71(c)					
01010			20.406(a)(1)(ii)			50.36(e)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
			20.406(a)(1)(iii)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(A)								
			20.406(a)(1)(iv)			50.73(a)(2)(iii)			50.73(a)(2)(viii)(B)								
			20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)

NAME
T. A. KRIESEL, TECHNICAL - PHYSICAL SCIENCES DEPARTMENT

TELEPHONE NUMBER

AREA CODE

6116 416151-159011

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	
B	I L L	M I O I N	E 1 0 1 7 1 0	N			I	I I I I	I I I I		
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SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR			
			0	7	0	9	8	4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON MARCH 11, 1984 AT 1149 HOURS, WITH THE REACTOR COOLANT SYSTEM IN MODE 5, THE UPPER CONTAINMENT AREA RADIATION MONITOR (VRS-2201) RECEIVED A HIGH ALARM THAT RESULTED IN AUTOMATIC ACTUATION OF THE ENGINEERED SAFETY FEATURE CONTAINMENT ISOLATION SYSTEM - TECHNICAL SPECIFICATION 3.3.2.1, TABLE 3.3-3, ITEM 3c (NOT REQUIRED IN MODE 5) - PURGE AND EXHAUST ISOLATION. THE CONTROL TERMINAL PRINTOUT WAS +0.00E+00 mR/HR AT THE TIME OF THE HIGH ALARM. THE HIGH ALARM WAS CLEARED ON VRS-2201 BY GOING TO ALARM "OFF" AND THEN BACK TO ALARM "ON" POSITION. THIS RETURNED VRS-2201 TO NORMAL STATUS AND PURGE WAS THEN MANUALLY RESTARTED.

THE IDENTICAL SEQUENCE OUTLINED ABOVE OCCURRED FIVE ADDITIONAL TIMES; 1849, 2009 AND 2139 HOURS ON MARCH 11, 1984, AND 0047 AND 0937 HOURS ON MARCH 17, 1984.

AT THE TIMES OF ALL THE EVENTS, THE CONTROL TERMINAL PRINTOUT READING WAS +0.00E+00 mR/HR WHICH IS AN INDICATION THAT NO HIGH AREA RADIATION LEVELS ACTUALLY EXISTED. IT IS THEREFORE SUSPECTED THAT THESE OCCURRENCES ARE THE RESULT OF EQUIPMENT MALFUNCTIONS POSSIBLY DUE TO SOFTWARE PROBLEMS. THE MANUFACTURER HAS BEEN NOTIFIED OF THESE OCCURRENCES AND RESOLUTION IS CURRENTLY UNDER EVALUATION. UPON THE RESOLUTION OF THIS PROBLEM AN UPDATED LER WILL BE SUBMITTED.

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PDR ADOCK 05000316
S PDR



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

April 6, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 84-003-0

Sincerely,

E.L. Townley

FOL W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
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