NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Salem Generating Station - Unit 1	0 5 0 0 0 2 7 2	1 OF 0 3

TITLE (4)
Containment Systems - Type B and C Leak Rate - Out of Specification

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 3	0 6	8 4	8 4	— 0 0 7	— 0 0	0 4	0 5	8 4		0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
5	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
J. L. Rupp	AREA CODE 6 0 9 — 3 3 9 — 4 3 0 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	L F	V	V 0 8 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		0 8	0 1	8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 6, 1984, during a refueling outage, PSE&G commenced routine periodic leak rate testing of Type C components; at which time, a check valve (1SA119) located inside containment and in the containment Station Air supply line, exhibited a leakage rate greater than the capacity of the leak rate test equipment. Because of this, the actual leak rate and the total combined leakage of all Type B and C components could not be verified to be within specification. The Containment Station Air Supply Manual isolation valve (1SA118) is a normally closed valve, and its leak rate was determined to be within the limits allowed by the Technical Specifications. The combined leakage rate of all Type B and C components had previously been demonstrated within specification during the last routine leak rate testing performed during the period of October 1982 to February 1983. Type B and C leak rate testing is still in progress. As required by the Technical Specifications, 1SA119, and all other containment isolation valves which are identified as exceeding allowable limits, will be repaired and retested prior to changing operating modes, and exceeding two-hundred degrees. A supplement to this report will be issued, at the completion of testing, identifying these components, the cause of their failure, and any corrective action taken.

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes are indentified in the text as [XX].

IDENTIFICATION OF OCCURRENCE:

Containment Systems - Type B and C Leak Rate - Out-of-Specification

Event Date: 03/06/84

Report Date: 04/05/84

This report was initiated by Incident Report No. 84-040

CONDITIONS PRIOR TO OCCURRENCE:

Mode 5 - Rx Power 000 % - Unit Load 0000 MWe

DESCRIPTION OF OCCURRENCE:

On March 6, 1984, during a refueling outage, while performing routine leak rate testing of Type C components, 1SA119 exhibited a leakage rate greater than the maximum range of the leak rate test equipment (20,000 sccm). 1SA119 is a check valve (located inside of containment) in the Station Air [LF] supply line to the containment. 1SA118 (in series with 1SA119, and located outside of containment) is the Containment Station Air Supply Manual Isolation Valve. Leak rate testing of 1SA118 indicated that the leakage (through 1SA118) was within the limits allowed by the Technical Specifications. 1SA118 is a normally closed valve; opened only when Station Air is required inside of containment. 1SA118 was opened (to supply air to the containment) at various times during operation in Modes 1 through 4. Because of this, and the fact that the actual leak rate of 1SA119 could not be ascertained, it was not possible to demonstrate that the total Type C leakage was less than 0.60 La as required by Technical Specifications Limiting Condition for Operation 3.6.1.2.b (applicable only in Modes 1 through 4), during the periods when 1SA118 was opened.

At the present time, Technical Specifications do not list 1SA119 as a containment isolation valve which requires periodic leak rate testing. As a conservative measure, Field Directive S-C-A900-MFD-068 identifies this valve (among others) as a containment isolation valve, and directs the leak rate testing to be performed. In the future, a License Change Request will be submitted, requesting that these valves be added to the Technical Specification list of containment isolation valves. Until then, PSE&G will consider this valve as a containment isolation valve.

APPARENT CAUSE OF OCCURRENCE:

1SA119 has not yet been repaired, and therefore the cause of the greater than allowed leakage has not been determined.

ANALYSIS OF OCCURRENCE:

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of forty-seven (47) psig.

Technical Specification Action Statement 3.6.1.2 requires:

> With either the measured overall integrated containment leakage rate exceeding 0.75 La, or with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 La, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System [AB] temperature above two-hundred (200) degrees.

Leakage rate limits during the performance of periodic tests incorporate conservatism to account for possible degradation of the leakage barriers between testing. The combined leakage rate of all Type B and C components had previously been demonstrated within specification during the last leak rate testing performed during the period of October 1982 to February 1983. Although 1SA118 was opened at various times during operation in Modes 1 through 4, Station Air pressure is approximately 100 psig to 110 psig. Containment pressure is normally maintained between minus 0.1 and plus 0.3 psig. Even during an accident condition, maximum containment pressure would be 47 psig, which is well below normal Station Air pressure. Therefore, the possibility of leakage out of this connection is minimum. This event involved no undue risk to the health or safety of the public. Due to the loss of redundancy in containment barriers, the event is reportable in accordance with the Code of Federal Regulations, 10CFR 50.73(a)(2)(ii). This report is required within thirty (30) days of discovery.

CORRECTIVE ACTION:

Type B and C leak rate testing is still in progress. All components exceeding the allowable leakage limits will be repaired and retested prior to changing operating modes, and exceeding two-hundred (200) degrees. A supplement to this report will be issued, at the completion of testing, identifying these components, the cause of their failure, and any corrective action taken.

General Manager-
Salem Operations

JLR:tns

SORC Mtg 84-038

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

April 5, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

SALEM GENERATING STATION
LICENSE NO. DPR-70
DOCKET NO. 50-272
UNIT NO. 1
LICENSEE EVENT REPORT 84-007-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR 50.73(a)(2)(ii). This report is required within thirty (30) days of discovery.

Sincerely yours,

J. M. Zupko, Jr.
General Manager -
Salem Operations

JR:kll

CC: Distribution

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