

NRC FORM 366
(12-81)U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011
EXPIRES 4-30-82CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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CONT

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REPORT SOURCE														DOCKET NUMBER										EVENT DATE										REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal startup at 1630, while troubleshooting the source of a

0 3 Shutdown Sequence Initiated Signal, maintenance personnel reported #12

0 4 Emergency Diesel Generator (EDG) to #21 4KV Bus out of service (T.S.3.8.6.1).

0 5 During the subsequent repair at 1639, output breaker to #14 4KV Bus was

0 6 racked out, which restored service to #21 4KV Bus. #11 and #21 EDG's

0 7 remained operable during the event. #12 EDG was returned to full service

0 8 at 1850. No similar events had been reported.

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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 #12 EDG output breaker to #14 4KV Bus had an auxiliary contact stuck

1 1 closed. GE Co. device MJ (GE type SBM 10AX012G3) shaft brearing was

1 2 found bound to switch housing. Replaced switch with like spare. Per-

1 3 manent facility change deenergized heaters in 4KV cubicles to eliminate

1 4 excessive temperature buildup. No further action is required.

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7	8	9	10	11	12	13	14	15	16	17	18																																																
FACILITY STATUS												% POWER												OTHER STATUS												METHOD OF DISCOVERY												DISCOVERY DESCRIPTION											
ACTIVITY CONTENT RELEASED OF RELEASE												AMOUNT OF ACTIVITY												LOCATION OF RELEASE																																			
Z												Z												NA												NA																							
PERSONNEL EXPOSURES NUMBER												TYPE												DESCRIPTION																																			
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NAME OF PREPARER G.S. Pavis/P.G. Rizzo/R.Androsik PHONE: 301-260-4466/4786/4986

LER NO. 81-58/3X
DOCKET NO. 50-317
LICENSE NO. DPR-53
EVENT DATE 07-21-81
UPDATE REPORT DATE 03-30-84
ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

Breaker No. 152-1406 which is the No. 12 EDG output breaker to No. 14 4KV Bus, was found to have an auxiliary contact switch (G.E. Device MJ) which bound in a position indicating the breaker was shut. Closed MJ contacts maintained the breaker shut signal to the Engineered Safety Features Actuation System which, in turn, gave erroneous annunciation of a Shutdown Sequence Initiated Signal. Other contacts of the switch, interlocked with the No. 12 EDG output breaker to No. 21 4KV Bus, would have prevented that breaker from closing.

Maintenance personnel racked out breaker No. 152-1406 from its cubicle (a General Electric Metal-Clad Switchgear Type MC-4.76). This action operated the breaker position switch, restoring No. 12 EDG service to No. 21 4KV Bus and removing service to No. 14 4KV Bus.

The MJ device was found to have its rear shaft bearing binding to the case material. High temperatures within the cubicle may have contributed to a change in bearing-to-housing clearance. (The switch is a G.E.Co. Type SBM P.N. 10AX012G3). The switch was replaced with a like spare and all 4KV cubicles' heaters were checked deenergized to ensure no heat build-up due to that source. During this review, no excessive temperatures were noted.

Since the date of this event, other switches have exhibited the same failure characteristic; however, the units replaced exhibited no similar failures with heaters deenergized.

Based on the manufacturer's original recommendations, to deenergize the 4KV cubicles' heaters, an engineering study determined that deenergizing the heater circuits would have no effect on the proper operation of the circuit breakers. A permanent Facility Change was initiated to deenergize these heaters and no further action is required.

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

March 30, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Docket No. 50-317
License No. DPR-53

Dear Sirs:

The attached supplemental report to LER 81-58/3X, Revision 1, is being sent to you as required by 10 CFR 50.73.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L.B. Russell

L. B. Russell
Plant Superintendent

R.A.
LBR:RA:mst

cc: Dr. T. E. Murley
Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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