

CONTROL BLOCK:

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	C	B	E	P	1	2	0	0	-	0	0	0	0	0	0	3	4									4			5
7	8	9	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE					30	57	CAT	58

CON'T

0	1
7	8

REPORT SOURCE

1	6	0	5	0	-	0	3	2	5	7	0	9	2	2	8	1	8	0	3	2	9	8	4	9
60	61	DOCKET NUMBER					62	63	EVENT DATE					74	75	REPORT DATE					80			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During a reactor startup, 1B reactor recirculation pump tripped concurrent with the receipt of the pump ATWS high reactor pressure/low level trip annunciator. At the time of the event reactor level was normal and pressure was at 0 psig. Immediately after the pump trip all control rod withdrawal was stopped. This event did not affect the health and safety of the public.

07 _____
08 _____ Technical Specifications 3.4.1.1, 6.9.1.9b _____

SYSTEM CODE C B 11		CAUSE CODE X 12		CAUSE SUBCODE Z 13		COMPONENT CODE I N S T R U 14				COMP. SUBCODE Y 15		VALVE SUBCODE Z 16					
EVENT YEAR 8 1 21		SEQUENTIAL REPORT NO. 0 5 3 24		OCCURRENCE CODE 0 3 28		REPORT TYPE L 30		REVISION NO. 1 32									
ACTION TAKEN C 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 22		ATTACHMENT SUBMITTED N 23		NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER R 3 6 9 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 The event is attributed to a spurious trip signal from ATWS low water level instrument

1 1 B21-LTM-NO24B-2; however, an evaluation of the subject instrument by the manufacturer

1 2 failed to reveal any problems. NO24B-2, Model No. 510DU216026A010, was replaced and

1 3 returned to service. No further action is planned regarding this event.

1	4																	80		
7	8	9																	80	
FACILITY STATUS			% POWER			OTHER STATUS			30	METHOD OF DISCOVERY			DISCOVERY DESCRIPTION						32	
1	5	C	28	0	0	0	29	NA			44	A	31	Operational Event						80
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25		

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

NA NA

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0 0 0	(37) Z (38)	NA					

7		8		9		11		12		13	
PERSONNEL INJURIES											
NUMBER						DESCRIPTION					
1	4	0	0	0	40	NA					

[illegible]

7 3 9 10 2
PDR ADOCK 05000325
S PDR
NRC USE ONLY

7	8		9	10	65	69	
2	9		<div style="border: 1px solid black; width: 1em; height: 1em; display: flex; align-items: center; justify-content: center;"><div style="font-size: 0.8em;">N</div><div style="margin-left: 2px;">(44)</div></div>	DESCRIPTION	NA		

NAME OF PREPARED

M. J. Pastva, Jr.

2HON[®]

919-457-9521



Carolina Power & Light Company

84 MAR 2 10:00

Brunswick Steam Electric Plant

P. O. Box 10429

Southport, NC 28461-0429

March 29, 1984

FILE: B09-13510C

SERIAL: BSEP/84-0869

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
SUPPLEMENT TO LICENSEE EVENT REPORT 1-81-53

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/jlh/LETJH1

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

OFFICIAL COPY

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