

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
James A. FitzPatrick Nuclear Power Plant

DOCKET NUMBER (2)

0 5 0 0 0 3 3 1 OF 0 3

TITLE (4)

Heat-up Rate Exceeded Technical Specification Limit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)					
0	3	1	3	8	4	8	4	0	0	8	0	5	0	0	0
0	3	1	3	8	4	0	0	0	3	3	0	5	0	0	0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																								
N	<table border="1"><thead><tr><th>20.402(b)</th><th>20.406(c)</th><th>50.73(a)(2)(iv)</th><th>73.71(b)</th></tr></thead><tbody><tr><td>20.406(a)(1)(i)</td><td>50.36(c)(1)</td><td>50.73(a)(2)(v)</td><td>73.71(c)</td></tr><tr><td>20.406(a)(1)(ii)</td><td>50.36(c)(2)</td><td>50.73(a)(2)(vi)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td></tr><tr><td>20.406(a)(1)(iii)</td><td>50.73(a)(2)(i)</td><td>50.73(a)(2)(viii)(A)</td><td></td></tr><tr><td>20.406(a)(1)(iv)</td><td>50.73(a)(2)(ii)</td><td>50.73(a)(2)(viii)(B)</td><td></td></tr><tr><td>20.406(a)(1)(v)</td><td>50.73(a)(2)(iii)</td><td>50.73(a)(2)(ix)</td><td></td></tr></tbody></table>	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)	20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)	20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)		20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	
20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)																						
20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)																						
20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)																						
20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)																							
20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)																							
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)																							

LICENSEE CONTACT FOR THIS LER (12)
NAME
Douglas J. Lindsey

TELEPHONE NUMBER

AREA CODE

3 1 5 3 4 2 - 3 8 4 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/>	<input checked="" type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During routine start-up operations, the 100°F per hour heat-up rate limit of Technical Specification 3.6.A.1 was exceeded by approximately 4°F during a one (1) hour period. The heat-up rate, prior to and after the period in question, was well below the Technical Specification limit. Personnel error was the cause of the event. An engineering evaluation was performed and has demonstrated that the event did not have any effect on reactor vessel life. Personnel involved were counselled and alarm points have been incorporated in the plant process computer to alarm at fifteen (15) minute intervals when the heat-up rate limit is being approached. The event has been incorporated into the licensed operator requalification program.

8404040167 840330
PDR ADOCK 05000333
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
James A. FitzPatrick Nuclear Power Plant	0500033384	—	008	—	002	OF 03

TEXT (If more space is required, use additional NRC Form 366A (17))

During a normal Reactor Start-up, the heat-up rate exceeded the Technical Specification limit of 100°F per hour by approximately 4°F.

The Reactor Operator assigned to the 09-5 Panel conducting the reactor heat-up was observing the nuclear instrumentation and withdrawing control rods to increase reactor power while attempting to correlate the heat-up rate with the IRM trace and receiving 15 minute updates from the Senior Nuclear Operator on the actual heat-up. During this period of time, reactor temperature was also recorded on the trend recorder which was indicating that the rate of change of temperature was increasing.

The Reactor Operator conducting the heat-up also observed that reactor water level was increasing and required constant attention.

The Senior Nuclear Operator at this time was relaying heat-up rate information from the process computer to the Reactor Operator conducting the heat-up. The Senior Nuclear Operator stated that during this period of time, there was a great deal of activity in the control room due to vessel level control and that just previous to the period of time that the heat-up rate was exceeded, other operational activities were occurring. The Senior Nuclear Operator, although relaying the heat-up information off the process computer, was reading the data off the computer printout incorrectly.

The Nuclear Control Operator, at this time, was also focused in on reactor water level control.

The Shift Supervisor was heavily involved with start-up activities and was making phone calls to the Reactor Analyst Supervisor and the Assistant Operations Superintendent. He assumed that the Assistant Shift Supervisor was monitoring the start-up during this period of time.

The Assistant Shift Supervisor, although directly supervising the start-up and heat-up prior to exceeding the heat-up rate, also got involved in additional plant activities and expressed that prior to exceeding the heat-up rate everything indicated the heat-up was going well.

Personnel error was the cause of the event. The Reactor Operator conducting the heat-up was relying on the Senior Nuclear Operator to provide heat-up rate information. The Senior Nuclear Operator was transmitting the heat-up information from the process computer to the Reactor Operator conducting the heat-up incorrectly. The control room operator was focused in on reactor water level and the overall big picture of plant start-up was temporarily lost. This is primarily a result of inexperience in conducting reactor start-up.

In addition, the event would have been mitigated had the Shift Supervisor or Assistant Shift Supervisor been directly supervising the heat-up at the time of the event.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
James A. FitzPatrick Nuclear Power Plant	0500033384	-	008	-	00	03 OF 03	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

An engineering evaluation was performed and has demonstrated that the event did not have any effect on reactor vessel life. Personnel involved were counselled and alarm points have been incorporated in the plant process computer to alarm at fifteen minute intervals when the heat-up rate limit is being approached. The event will also be discussed between all the operating shifts and the Operations Superintendent during the operating shifts training week. The event has been incorporated into the licensed operator requalification program.

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Lycoming, New York 13093
315 342.3840



Corbin A. McNeill, Jr.
Resident Manager

March 30, 1984
JAFF 84-0350

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

REFERENCE: DOCKET NO. 50-333 Licensee Event Report: 84-008-00

Dear Sir:

We have enclosed the referenced Licensee Event Report in accordance with
10CFR50.73

If there are any questions concerning this report, please contact Mr. Douglas J.
Lindsey at (315) 342-3840, Extension 302.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'by dr. R. McNeill'.

CORBIN A. McNEILL, JR.
RESIDENT MANAGER

CAM:DJL:dmh
Enclosure

CC: USNRC, Region I (1)
INPO Records Center, Atlanta, Georgia (1)
Internal Power Authority Distribution
NRC Resident Inspector
Document Control Center
LER/OR File

IE22
11