

NRC FORM 366
(12-81)
10 CFR 50U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 M A P P I S I 2 0 0 0 - 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 1 1 1 4 5
7 8 9 LICENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 36

CONT

01 REPORT SOURCE L 6 0 5 0 0 2 9 3 7 0 7 2 4 8 0 8 0 3 0 8 8 4 9
7 8 DOCKET NUMBER 45 46 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On July 24, 1980, while performing surveillance test number 8.7.4.3 entitled,
03 "Test Isolation Valves Except MSIV's" isolation valve 5065-17 and 22 were de-
04 clared inoperable. These valves required 20 seconds versus a five second limit
05 to close. The valves control two of seven containment sample lines of the
06 Oxygen Analyzer. This event did not impact the health and safety of the public.

07
08

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
S D 11 E 12 B 13 V A L V E X 14 D 15 D 16
7 8 9 10 11 12 13 14 15 16 17 18 19 2017 LER/RO REPORT NUMBER 8 0 21 22
18 0 4 1 23 24
19 0 3 25 26
20 X 27 28
21 29
30 31
32 33ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
B 18 Z 19 Z 20 Z 21 0 0 0 0 N 23 N 24 N 25 A 4 9 9 26
33 34 35 36 37 38 39 40 41 42 43 44 45

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The valves were electrically disabled in the closed position per Tech Spec until
11 repairs were made. Cause of the valve failure was determined to be valve limits
12 being out of adjustment. Valve limits were adjusted and the valves were exercised
13 to provide correct operation. A search of records shows no similar failures of
14 the subject valves since 7/24/80.

15 FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32
E 28 1 0 0 0 29 NA B 31 Surveillance Testing
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 3216 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
Z 33 Z 34 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 3217 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39
0 0 0 0 37 Z 38 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 3218 PERSONNEL INJURIES NUMBER DESCRIPTION 41
0 0 0 0 40 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 3219 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43
Z 42 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 3220 PUBLICITY ISSUED DESCRIPTION 45
N 44 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 328404030507 840308
PDR ADOCK 05000293
S PDR

NRC USE ONLY

NAME OF PREPARER P.J. Hamilton

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WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

March 8, 1984

BECO. #84-039

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-293
License DPR-35

Dear Sir:

The attached Update Licensee Event Report 80-41/03X-1, "O₂ Isolation Valves 5065-17 and 5065-22", is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specification 6.9.B.2.b.

If there are any other questions on this subject please do not hesitate to contact me.

Respectfully submitted,

W D Harrington
William D. Harrington

WDH/mg

Enclosure: LER 80-41/03X-1

cc: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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