

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
R. E. Ginna Nuclear Power Plant, Unit No. 1	0 5 0 0 0 2 4 4	1 OF 0 4

TITLE (4)

Inoperable S.I. Accumulators

EVENT DATE (6)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																								
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)																		
0	2	19	8	4	8	4	-	0	0	1	-	0	0	0	3	1	9	8	4						0	5	0	0	0					
0	2	19	8	4	8	4	-	0	0	1	-	0	0	0	3	1	9	8	4						0	5	0	0	0					

OPERATING MODE (8)		N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 9 8		20.402(b)		20.406(e)		X		50.73(a)(2)(iv)		73.71(b)			
		20.406(a)(1)(i)		50.38(c)(1)				50.73(a)(2)(v)		73.71(c)			
		20.406(a)(1)(ii)		50.38(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 386A)			
		20.406(a)(1)(iii)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(A)							
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)							
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)							

LIGENEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER				
G. F. Larizza, Operations Manager	<table border="1"> <tr> <td>AREA CODE</td> <td></td> </tr> <tr> <td>3 1 5</td> <td>5 2 4 - 4 4 4 6</td> </tr> </table>	AREA CODE		3 1 5	5 2 4 - 4 4 4 6
AREA CODE					
3 1 5	5 2 4 - 4 4 4 6				

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	B Q	- A C C	D 1 D 0	Y		X	L D	- C N V	F 1 3 0	Y	
X	L K	- P C V	C 6 3 5	Y		X	L K	- - R V	C 7 1 1	Y	

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)		<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)			
---	--	--	-------------------------------	--	--	--

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Due to existing nitrogen system leakage, the S.I. Accumulators were being filled at intervals of twice per shift. It was during a routine nitrogen fill that on February 18, 1984, at 2205 hours while operating at approximately 98% power, the Control Room Operator noticed accumulator pressure, instead of increasing and clearing the low pressure alarm, had in fact decreased to below the Technical Specification limit. The "B" S.I. Accumulator was declared inoperable and within the next hour a unit shutdown was commenced as required by plant Technical Specification. At 0015 hours on February 19, 1984, the "A" S.I. Accumulator was also declared inoperable. An unusual event was declared at 0020 hours. The Control Room personnel attempted to makeup nitrogen to both accumulators but instead of pressure increasing, the pressure kept decreasing. Verification of valve lineups were performed in the Auxiliary Building and found no indication of a problem. Subsequent containment entry revealed HCV-945 (in the closed position) to be leaking nitrogen by the seat. A manual valve downstream was closed and cap installed thus terminating the nitrogen pressure loss from accumulators. The accumulators were declared within Technical Specification pressure requirements, load reduction terminated at 0425 hours and unusual event terminated at 0430 hours on February 19, 1984.

B404030466 B40319
PDR ADCK 05000244
S PDR

IE22

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 9/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
R.E. Ginna Nuclear Power Plant, Unit No. 1	05000244	84	001	00	02	OF 04

TEXT (If more space is required, use additional NRC Form 386A's) (17)

Due to existing nitrogen system leakage, (relief valve 830A on "A" S.I. Accumulator) the S.I. Accumulators were being filled at intervals of twice per shift. It was during a routine nitrogen fill that on February 18, 1984 at 2205 hours while operating at approximately 98% power, the Control Room Operator noticed accumulator pressure instead of increasing and clearing the low pressure alarm had in fact decreased to below Technical Specification limit. The line-up and all valves relative to making up nitrogen to the accumulators outside the containment were checked and no problems found. The "E" S.I. Accumulator was declared inoperable due to low pressure, thus entering a Technical Specification limiting condition for operation that requires to correct the problem or within one hour start shutting down and be in hot shutdown in 6 hours and less than 350°F Tavg. in an additional 6 hours. The unit shutdown commenced at 2300 hours.

On February 19, 1984, at 0015 hours, the "A" S.I. Accumulator pressure dropped below the Technical Specification setpoint and was also declared inoperable. At 0020 hours, an Unusual Event was declared. Personnel entering containment between 0001 hours and 0030 hours could not find the leak. The pressure drop on the accumulators at this time had either stopped or had slowed, both accumulators were between 650 and 675 psig.

A second containment entry was then made at about 0330 hours. At this time, the accumulators vent, HCV-945 was found to be leaking through. Manual Isolation Valve 945A was closed and a cap placed on the end of this line, this terminated the nitrogen leak.

At 0425 hours, both accumulators were charged to greater than the Technical Specification required pressure and were declared operable and the load reduction terminated. At 0430 hours the unusual event was terminated and power increase to 98% commenced.

Both accumulators lost nitrogen while charging because the Charging Vent Valve (834B, "E" S.I. Accumulator) was held in the open position because of apparent binding. If it had failed in the closed position, makeup to the "E" S.I. Accumulator would not be possible. With this valve open and when the Loop "B" Accumulator is charged both accumulators are tied together. This is the reason that when HCV-945 started leaking through, the Loop "A" Accumulator was the first to be out of specification and subsequently the Loop "B" Accumulator, as nitrogen make-up was started.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
R.E. Ginna Nuclear Power Plant, Unit No. 1	0 5 0 0 0 2 4 4 8 4	—	0 0 1	—	0 0 0	3	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

(CONTINUED)

Subsequent investigation revealed that the reason HCV-945 started leaking is that the current to pressure transducer that converts controller signal to an air signal for the valve positioner was found out of calibration. The output had shifted high enough to start opening the valve with the controller set at the full closed position. The cause for the shift was dirt in the pneumatic portion of the unit.

The corrective action included cleaning and calibrating I/P transducer and the valve positioner, replacement of I/P transducer airset (including filters). The preventative maintenance schedule will be reviewed for similar valves.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)							
		YEAR	SEQUENTIAL NUMBER			REVISION NUMBER									
R. E. Ginna Nuclear Power Plant, Unit No. 1	0 5 0 0 0 2 4 4	8	4	-	0	0	1	-	0	0	0	4	OF	0	4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	LK	- FIC V C6 35	Y	



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700



March 19, 1984

Dr. Thomas E. Murley, Regional Administrator
U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: LER 84-001, Inoperable Safety Injection Accumulators

R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Dr. Murley:

In accordance with 10 CFR 50.73, Licensee Event Report System item (2)(a)(v), "any event or condition that alone could have prevented the fulfillment of the safety function of structures or systems that are needed to:

- (a) Shutdown the reactor and maintain it in a safe shutdown condition;
- (b) Remove residual heat;
- (c) Control the release of radioactive material;
- (d) Mitigate the consequences of an accident;"

the attached Licensee Event Report LER 84-001 is hereby submitted.

Very Truly Yours,

Roger W. Kober
Roger W. Kober

Attachment

xc: Document Control Desk (1)

IE22
11