

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

PHONE: (315) 349-2467

Event Description and Probable Consequences

An NRC inspection of NUREG 0737, shielding review, revealed that the four containment spray test valves (EP Numbers 80-40, 80-41, 80-44 and 80-45) may not meet the seismic criteria due to the installation of air operators. These operators were installed without proper seismic analysis.

Cause Description and Corrective Actions

The cause of this problem was the failure to perform a seismic analysis in the design of the modification. On July 11, 1983, the air operators on valves 80-44 and 80-45 were removed to restore one containment spray system to an acceptable configuration for seismic operability of one system. The following day, July 12, 1983, the air operators on valves 80-40 and 80-41 were then removed to place both containment spray systems in an acceptable condition for seismic operability and to comply with Technical Specification 3.3.7. A seismic analysis of the containment spray test/alternate torus cooling lines affected by the installation of the subject valve operators was performed by MPR Associates and Niagara Mohawk Power Corporation to determine the seismic acceptability of these systems. The results of this analysis concluded that the stresses in the containment spray valves with operators and piping due to the combination of normal loads and the specified seismic loads are within Nine Mile Point Unit No. 1 FSAR allowable values.

On this basis, we have reinstalled the valve operators without any modification to the piping system, valves or their supports.

March 8, 1984

Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

Re: Docket No. 50-220
LER 83-20, Rev. 1

Dear Dr. Murley:

In accordance with Nine Mile Point Nuclear Station Unit 1 Technical Specifications, we hereby submit the following Licensee Event Report:

83-20

Rev. 1

which is being submitted in accordance with Section 6.9.2a(9), Performance of structures, systems or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses, in the safety analyses, in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

This report was completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours,

Thomas E. Lemppes

Thomas E. Lemppes
Vice President
Nuclear Generation

TEL/RCN/jkr

Attachments (3 copies)

cc: Director, Office of I&E (30 copies)

Director, Office of MIPC (3 copies)

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