

NRC FORM 366 (12-81) 10 CFR 50			U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT			APPROVED BY OMB 3150-0011		
CONTROL BLOCK:			(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)					
P A S E S 1			0 0 - 0 0 0 0 0 - 0 0			4 1 1 1 1		
L I C E N S E E C O D E			L I C E N S E N U M B E R			L I C E N S E T Y P E		
CON'T			R E P O R T S O U R C E			D O C K E T N U M B E R		
0 1			L 0 5 0 0 0 3 8 7			0 4 1 4 8 3		
R E P O R T S O U R C E			D O C K E T N U M B E R			E V E N T D A T E		
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES			During the implementation of a plant modification to remove the internals of a Pacific check valve in the Emergency Service Water System to correct a recognized deficiency in those valves, the valve disc was found lodged in the pipe and various small valve parts were missing. There were no adverse consequences in that adequate system flow was achieved each time the system has been started.					
SYSTEM CODE			CAUSE CODE			COMP. SUBCODE		
W G			B B			C A		
L E R / R O R E P O R T N U M B E R			S E Q U E N T I A L R E P O R T N O .			O C C U R R E N C E C O D E		
8 3			0 6 6			0 3		
A C T I O N T A K E N			F U T U R E A C T I O N			E F F E C T O N P L A N T		
Z			Z			Z		
HOURS			ATTACHMENT SUBMITTED			PRIME COMP. SUPPLIER		
0 0 0 0			Y			A		
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS			Check valve wear was caused by extensive system run-time during the resolution of the ESW water hammer problem encountered last year. A generic problem with other Pacific check valves has been identified and a program developed for their testing.					
FACILITY STATUS			% POWER			OTHER STATUS		
B			0 0 0			NA		
ACTIVITY CONTENT RELEASED OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE		
Z			Z			NA		
PERSONNEL EXPOSURES NUMBER			TYPE			DESCRIPTION		
0 0 0			Z			NA		
PERSONNEL INJURIES NUMBER			DESCRIPTION					
0 0 0								
LOSS OF OR DAMAGE TO FACILITY TYPE			DESCRIPTION					
Z								
PUBLICITY ISSUED DESCRIPTION								
N								
NAME OF PREPARER			PHONE			NRC USE ONLY		
L.A. Kuczynski			(717) 542-2181			X3759		

ATTACHMENT

LER # 83-066/03X-1

Pennsylvania Power & Light Company
Susquehanna Steam Electric Station
Docket Number: 50-387

A plant modification was authorized to eliminate the abnormal wear concern on various Pacific check valves in the Emergency Service Water (ESW) system. This modification includes removing four affected check valves and replacing them with butterfly valves and removing the internals of two other check valves. It was during the internals removal of one of the check valves that the condition which precipitated this LER was discovered. When the valve was dis-assembled, the valve disc was found lodged in the pipe and the cotter pin and nut were missing. Engineering evaluation has concluded that the cotter pin does not provide a hazard to system operation. The nut is large enough to cause the ESW bypass valve to malfunction. An inspection of a portion of the ESW piping was performed and the nut was not found.

Other check valves which are subject to similar wear conditions as the one which failed will be tested and monitored under the Susquehanna Inservice Testing Program for Pumps and Valves.

1 | Work has been completed to tie the Unit 2 ESW and RHRSW piping into the line in which the nut may be contained. There was sufficient flow in the piping during the ESW/RHRSW full flow testing that, if the check valve nut were in the piping, it is reasonable to assume that it was flushed into the spray pond where it poses no threat to plant equipment. A hydrostatic test was performed on the line after the flow testing which verified the leak-tight integrity of the ESW bypass valve. Based upon the above, no further actions will be taken to locate the missing nut.



Pennsylvania Power & Light Company

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March 7, 1984

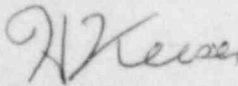
Dr. T.E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-066/03X-1
ER 100450 FILE 841-23
PLA-2124

Docket No. 50-387
License No. NPF-14

Dear Mr. Allan:

1 | Attached is updated Licensee Event Report No. 83-066/03X-1, which describes subsequent analyses regarding inspection to locate missing check valve nut in ESW piping.


H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

Attachment

cc: Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555

G.G. Rhoads
Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

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