



GULF STATES UTILITIES COMPANY

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March 29, 1984

RBG- 17,450

File Code G9.5, G9.8.6.2

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Denton:

River Bend Station Unit 1
Docket No. 50-458

On March 27, 1984, a conference call was held between Gulf States Utilities Company (GSU) and the Nuclear Regulatory Commission's (NRC) Accident Evaluation Branch (Mr. Larry Bell) to discuss River Bend Station's (RBS) March 1, 1984 submittal of calculational methodology for equipment qualification radiation environments. At the conclusion of the telephone conversation, Mr. Bell indicated that the methodology was acceptable with the following clarification.

Our previous submittal only addressed the calculational methodology used to evaluate radiation levels for accident conditions (i.e., a large break LOCA). The source terms used to determine the normal radiation levels are based on the fission product and activation product sources which represent design failed fuel conditions corresponding to an offgas release rate of 100 uCi/sec/MWt or 304,000 uCi/sec at 30 min decay. These values were obtained by scaling up from expected NUREG-0016, Revision 1 "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors", source terms. Additional discussion of these source terms can be found in FSAR Section 11.1.

The total integrated radiation dose used for equipment qualification corresponds to the normal radiation dose received at a particular location over the equipment design life plus the applicable accident dose. This is discussed in Section 3.11.5.2 of the River Bend Station FSAR.

Sincerely,

Eddie R. Grant

for J. E. Booker
Manager-Engineering
Nuclear Fuels & Licensing
River Bend Nuclear Group

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