

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)				PAGE (3)	
Sequoyah, Unit 1										0 5 0 0 0 3 2 7				1 OF 0 2	

TITLE (4)
Containment Building Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)											
0	2	2	7	8	4	8	4	—	0	1	7	—	0	0	0	3	2	7	8	4	0 5 0 0 0				
																		0 5 0 0 0							

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)				
5		20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	60.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	0 10 0	20.405(a)(1)(i)	60.36(e)(1)		60.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	60.36(e)(2)		60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	60.73(a)(2)(i)		60.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	60.73(a)(2)(ii)		60.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	60.73(a)(2)(iii)		60.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)										
NAME							TELEPHONE NUMBER			
Glenn Duggin, Compliance Section Engineer							AREA CODE			
							6	1	5	8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A high radiation alarm was actuated which caused a containment ventilation isolation (CVI) to occur. Investigation revealed that following the removal of the primary side manway cover of a steam generator, a small amount of airborne radiation escaped. An increase in the radiation level in lower containment was expected during this operation. No personnel were contaminated and no significant airborne radiation was detected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 1	0 5 0 0 0 3 2 7	8 4	— 0 1 7	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This LER involves two separate incidents. The first containment ventilation isolation (CVI) occurred at 0600C on 02/27/84 while unit 1 was in mode 5 (0% power, 180 psig, 105 degrees F) and was returned to service at 0624C on 02/27/84. The second CVI occurred at 1332C on 02/27/84 while unit 1 was in mode 5 (0% power, 180 psig, 105 degrees F) and was returned to service at 1351C on 02/27/84. All associated equipment and personnel responded and performed as expected during the CVI. The operator responded to the alarm (RM-90-130, -131) and determined that the alarm was caused by a small quantity of airborne radiation which occurred during the performance of a maintenance instruction to remove the primary side manway cover from the steam generator. The alarm was cleared, and the radiation monitor was reset. No personnel were contaminated, and no other unusual radiation was detected.

In the first incident, the manway cover from steam generator number two was removed, and in the second incident, the manway cover from steam generator number three was removed. The maintenance instruction for the manway cover removal includes requirements for the gases from the steam generator to go through a HEPA filtering system before being released into the containment building vent duct. The exhaust gases were filtered as required, but the HEPA filter will not stop noble gases from escaping. This release was expected and personnel were protected. During future performances of this activity, the setpoint of the radiation monitor will be temporarily revised upward to allow for expected increases in radiation.

There was no effect on public health or safety, and no plant safety margins were exceeded.

Previous occurrences - SQRO-50-327/84001; SQRO-50-327/84003; SQRO-50-327/84009; and SQRO-50-327/84012.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

March 27, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

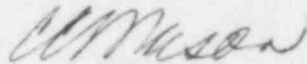
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84017

The enclosed licensee event report provides details concerning the containment building ventilation isolation following removal of the primary side manway from a steam generator. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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