

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

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March 22, 1984

BLRD-50-438/83-47
BLRD-50-439/83-39

U.S. Nuclear Regulatory Commission
Region II
Attn: Mr. James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

BELLEFONTE NUCLEAR PLANT UNITS 1 AND 2 - INCORRECT MODELING OF PROBLEMS IN
TPIPE - BLRD-50-438/83-47, BLRD-50-439/83-39 - FINAL REPORT

The subject deficiency was initially reported to NRC-OIE Inspector
Linda Watson on August 1, 1983 in accordance with 10 CFR 50.55(e) as
NCR BLN CEB 8304. This was followed by our interim report dated August 26,
1983. Enclosed is our final report.

TVA does not now consider the subject nonconforming condition adverse to
the safe operation of the plant. Therefore, we will amend our records to
delete the subject nonconformance as a 10 CFR 50.55(e) item.

If you have any questions, please get in touch with R. H. Shell at
FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills

L. M. Mills, Manager
Nuclear Licensing

Enclosure

cc: Mr. Richard C. DeYoung, Director (Enclosure)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center (Enclosure)
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

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ENCLOSURE

BELLEFONTE NUCLEAR PLANT UNITS 1 AND 2
INCORRECT MODELING OF PROBLEMS IN TPIPE
NCR BLN CEB 8304
BLRD-50-438/83-47, BLRD-50-439/83-39
10 CFR 50.55(e)
FINAL REPORT

Description of Deficiency

Isometric drawings for the reactor coolant (RC) drains, vents, and miscellaneous piping systems show that snubbers at nodal points 67 and 75 are to be 90° or perpendicular to the RC drain piping. The directions of supports at nodal points 67 and 75 or problem N4-1NK-C (unit 1) (N4-2NK-C) (unit 2) were not modeled in the TPIPE program consistent with the analysis isometric. Incorrect data using the snubbers skewed at 30° was input into TPIPE program which would affect the support design loads, the piping system movements and piping stresses.

Safety Implications

After revising the computer model to correctly represent the supports normal to the pipe (90 degrees off the axis of the pipe) and subsequent reanalysis, it was discovered that the loads actually decreased. The deficiency described above is considered to be an isolated event, no support design changes are needed and no conditions associated with this deficiency exist that would adversely affect the safety of the plant. Thus, we no longer consider 10 CFR 50.55(e) applicable.