

MONTHLY OPERATIONS REPORT

FEBRUARY 1984

Throughout the report period, the Oyster Creek Station remained shutdown for the current Refueling and Maintenance Outage. There is no fuel in the reactor.

Local Power Range Monitor (LPRM) replacement was completed except for LPRMs 28-41 and 20-49. LPRM 28-41 is stuck in the core and cannot be freed. A new LPRM could not be inserted in the LPRM housing 20-49. Maintenance and Construction and Plant Engineering have initiated an investigation to determine the cause. LPRM replacement was subsequently placed "on hold" to facilitate draining of the cavity for feedwater valve maintenance.

The plant experienced difficulty in draining the cavity due to the inability to establish a positive seal on the equipment pool blocks. After numerous unsuccessful attempts to seal the blocks, the decision was made to remove the equipment pool blocks, place the steam separator in the vessel and install the reactor head (ALARA considerations). The cavity and then the reactor was drained to below the feedwater sparger, at which time, repair to feedwater isolation valve V-2-36 was started. In conjunction, replacement of Cleanup System valve V-16-2 started along with repairs to V-16-1. By the end of the report period, repairs to V-2-36 were completed and reactor water level was raised to just below the reactor head flange.

On February 1, 1984, diesel generator No. 2 was released for its annual inspection. At the end of the report period, the plant was preparing to perform an operability test on the diesel generator (subsequently, D.G. No. 2 became operable on March 3, 1984).

Numerous problems were encountered with air compressor No. 2. The air compressor was out of service from January 4, 1984 to February 12, 1984 while waiting for special gasket material. On February 18, a recurring inter-stage relief problem was again experienced. The air compressor was returned to an operable status on February 27, 1984.

Dilution pump No. 2 overhaul was completed. However, various problems were encountered during acceptability testing. The major problem was identified as a failure of its drive motor (ground fault). The drive motor from No. 3 dilution pump was installed on No. 2 pump and tested successfully. No. 3 dilution pump has been released for its scheduled overhaul and inspection.

It was discovered that the dry tube for Intermediate Range Monitor (IRM) 12 is cracked. Plant Engineering, along with General Electric, is addressing the problem. The dry tube must be replaced prior to plant startup. An inspection of all IRM dry tubes has been scheduled for the week of March 12, 1984.

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The inspection, painting and hydro of "B" isolation condenser was completed. Work on "A" isolation condenser is presently in progress.

On February 26, 1984, Standby Gas Treatment System No. 2 failed its operability test on high HEPA filter differential pressure. The HEPA filters were replaced and a satisfactory D.O.P. test (efficiency test) was performed on February 29, 1984.

The inspection required by NRC Bulletin 84-04 ("Inspection of the Torus Vent Headers for Cracks") was satisfactorily completed.

Repairs to condensate transfer pump 1-1 were completed.

The cleaning of "C" hotwell was completed. The water in "A" and "B" hotwell has been transferred to "C" hotwell to facilitate cleaning and expansion joint replacement of A and B.

The main lube oil tank (Turbine) was refilled at the end of the report period. Presently, the plant is in the process of cleaning the oil prior to the start of setting the turbine controls.

The following Licensee Event Report was submitted during February 1984:

Reportable Occurrence No. 50-219/83-26/01T

Over the past several years the addition of lead, for the purpose of radiation shielding, on two of three fuel pool cooling heat exchangers has created a situation where the heat exchangers' foundation bolts would be overstressed during a seismic event. In addition, while investigating this situation, it was discovered that the original portions of the fuel pool cooling piping system are supported only by dead weight supports and therefore, may not be a seismic Class I system as stated in the station's "Facility Description and Safety Analysis Report" (FDSAR). The cause of the occurrence was attributed to lack of procedural controls in the past when applying lead shielding to piping systems. The exact cause of the discrepancy between the FDSAR and the installation specifications could not be determined but is believed to be due to lack of control of changes during the construction phase of the plant. The following corrective actions have been initiated:

1. A decontamination effort has been completed which reduced the levels of radiation in the vicinity of the heat exchangers. All lead will be removed prior to startup. If, for ALARA reasons, lead shielding is still required, a safety evaluation will be performed before adding any shielding.
2. A walk-down of the plant will be conducted to insure that similar situations do not exist which might interfere with the functioning of safety-related equipment. This will be completed prior to plant startup.
3. A seismic analysis has been conducted for the fuel pool cooling piping system in its present configuration and it was found that the original fuel pool cooling system is not seismic Class I; however, the augmented cooling system which was added as part of the fuel pool expansion described in Amendment 78 to the FDSAR is a seismic Class I system. To satisfy original licensing criteria, changes will be made to the return piping system to ensure a seismically qualified flow path can be established between the fuel pool and the seismically qualified portion of the cooling system. This will be accomplished prior to the next core offload. Seismic qualification will be based upon operational criteria consistent with ASME Section III, Division I, Appendix F. An assessment of the entire system will then be made to determine if further upgrading is appropriate.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February 1984

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE 3/5/84
 COMPLETED BY R. Baran
 TELEPHONE 971-4640

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	9192	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: February, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	696.0	1440.0	124368.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHTDWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHTDWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0.0	0.0	136224729
18. GROSS ELEC ENER (MWH)	0.0	0.0	46056905
19. NET ELEC ENER (MWH)	-1895	-3886	44281797
20. UT SERVICE FACTOR	0.0	0.0	66.5
21. UT AVAIL FACTOR	0.0	0.0	66.5
22. UT CAP FACTOR (MDC NET)	0.0	-0.4	57.4
23. UT CAP FACTOR (DER NET)	0.0	-0.4	54.8
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	6/11/84		

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #.50-219
UNIT.Oyster Creek #1
REPORT DATEMARCH 5, 1984
COMPILED BYDONALD V. NOTIGAN
TELEPHONE #609-971-4695

MONTH FEBRUARY, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0		

REFUELING INFORMATION - February, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: June 11, 1984

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Technical Specification Change Request No. 96 was submitted on August 31, 1982 for incorporation of GE assemblies into the Cycle 10 core.

Scheduled date(s) for submitting proposed licensing action and supporting information:

October 28, 1983 - The final supplement to the reload analysis, delineating the specific core configuration for Cycle 10 operation, was submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 0
(b) in the spent fuel storage pool = 1375

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800

Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.



GPU Nuclear Corporation

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609 971-4000
Writer's Direct Dial Number:

March 15, 1984

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

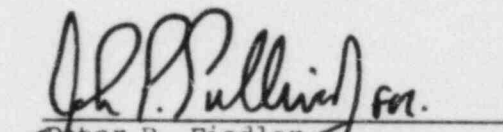
Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Michael Laggart at (609) 971-4643.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dam
Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

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