

SNUPPS

Standardized Nuclear Unit
Power Plant System

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March 26, 1984

SLNRC 84-0051 FILE: 0541/M-189
SUBJ: Preservice Inspection Relief
Request - Callaway Plant

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Nos. STN 50-482 and STN 50-483

Ref: SLNRC 84-0036, dtd. 2/24/84: Subj. as Above

Dear Mr. Denton:

The reference letter forwarded a partial relief request from preservice examination requirements for a number of component and piping system welds in Callaway Plant. The purpose of this letter is to advise NRC of an additional component weld requiring relief from ASME Section XI examination requirements. Specific data in support of this request are as follows:

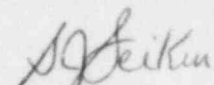
System:	Reactor Pressure Vessel
Category:	B-A
Component Description:	Top circumferential weld in the RPV Closure Head, I.D. 2-CH-103-101
Code Requirement:	Volumetric examination from both sides of weld per ASME Section XI, 1977 Edition, Summer 1978 Addenda
Areas for Relief:	35% of total weld volume
Basis for Relief:	Control Rod Drive mechanism penetrations, three lifting lugs welded directly onto 2-CH-103-101, and obstructing closure head shrouding, preclude complete volumetric examination. Removal of closure head shroud during ISI will require 500 manhours of effort, presenting considerable ALARA concerns and still would not permit complete weld coverage.
Alternate Testing:	None; Accept as is

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Q PDR

Boo!
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The above relief request was discussed via telecon with NRC (Hum, Holonich) on March 8, 1984 and the information provided herein is a result of this telecon. If additional information is needed in support of this request, please contact the undersigned.

Very truly yours,



S. J. Seiken, Manager
Quality Assurance

SJS/dck/3a2

cc: J. Neisler/B. Little, USNRC/CAL
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