



**Wisconsin Electric** POWER COMPANY  
231 WEST MICHIGAN, MILWAUKEE, WISCONSIN 53201

February 2, 1976

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Mr. Benard C. Rusche, Director  
Office of Nuclear Reactor Regulation  
U. S. NUCLEAR REGULATORY COMMISSION  
Washington, D. C. 20555

Dear Mr. Rusche:

LICENSEE EVENT REPORT 50-266/76-3  
POINT BEACH NUCLEAR PLANT  
LEAKAGE TESTS OF UNIT 1  
CONTAINMENT ISOLATION VALVES  
DURING REFUELING OUTAGE TESTING

This report follows an initial report to you of May 3, 1974, in which we committed to filing an abnormal occurrence on any valve which, when first subjected to a local leak test of 60 psig (air) during refueling outages, exhibited a leak rate in excess of 2000 cc/minute.

Nine valves fell into the above category during the Unit 1, Refueling 3 outage from November 16, 1975, to January 8, 1976.

Each valve leak test which exceeded the above administrative limit during its leak test was reported to Mr. Dwane Boyd, Office of Inspection and Enforcement - Region III, within 24 hours of that valve failing its test.

The test performance of the nine valves is described on the attached table.

Specific comments on each failure are as follows:

1. The body-to-bonnet bolted joint was found to be slightly open and the bolting was tightened.

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PDR ADOCK 05000266  
S PDR

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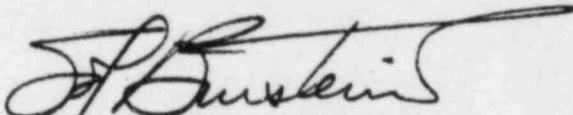
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- 2, 3, 4, 5,  
6, 8, and 9. Dirt was found on the valve seats and  
general cleaning and in some cases lapping  
the seats and discs was performed.
7. The valve required adjustment to permit  
the valve plunger to bottom on the seat.

Each of the above-reported valves is in series with  
a second containment isolation valve in that system. In each  
case, the second isolation valve demonstrated an acceptable  
level of leak tightness. It is, therefore, not considered that  
the leakage of the above-reported valves constituted a potential  
hazard to the health and safety of the public.

A full report of the Type "B" and "C" tests conducted  
during the Unit 1 third refueling shutdown will be filed  
separately as required by Section V.B of 10 CFR 50, Appendix J,  
following the completion of the next containment Type "A" test,  
presently scheduled for 1977.

Very truly yours,

  
Executive Vice President

Sol Burstein

Attachment

	<u>Penetration</u>	<u>Valve</u>	<u>Piping System</u>	<u>Initial Leakage cc/min</u>	<u>Date of Test</u>	<u>Final Leakage cc/min</u>	<u>Date of Test</u>
1.	9	2721	Reactor coolant drain tank and refueling cavity drain	16,000	12-24-75	10	12-30-75
2.	33A	IA-33A check	Instrument air to containment	32,000	12-07-75	674	12-22-75
3.	33B	IA-34A check	Instrument air to containment	13,000	12-07-75	350	12-22-75
4.	33C	Check valve	Service air to containment	50,000	12-21-75	0	12-28-75
5.	50	2045	"B" steam generator blowdown	2,000	12-08-75	2	12-23-75
6.	X1	3200B	R11/R12 supply line	175,000	12-10-75	0	01-06-76
7.	X2	3200A	R11/R12 return line	17,000	12-09-75	734	12-20-75
8.	15	755A check	Component cooling to "A" reactor coolant pump	10,000	11-29-75	17	12-08-75
9.	16	755B check	Component cooling to "B" reactor coolant	>200,000*	11-29-75	454	12-09-75

\*Could not measure with test rig.