



LWP-95-072

August 1, 1995

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

SUBJECT: Quad Cities Station Operating Report  
NRC Dockets (50-254 and 50-265)

Enclosed is the Radioactive Effluent Report for January through June 1995, for Quad Cities Nuclear Power Station.

A copy of this report will be furnished to the NRC Resident Inspector.

Sincerely,

COMMONWEALTH EDISON  
QUAD CITIES NUCLEAR POWER STATION

L. W. Pearce  
Station Manager

LWP/JGW/kjv

Enclosure

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# EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT

## Supplemental Information

Facility Quad Cities Nuclear Power Station

Licensee Commonwealth Edison Company

### 1. Regulatory Limits

#### a. For Noble Gases:

##### Dose rate

1. Less than 500 mrem/year to the whole body.
2. Less than 3000 mrem/year to the skin.

##### Dose Gamma Radiation

1. Less than or equal to 5 mrad/quarter.
2. Less than or equal to 10 mrad/year.

##### Beta Radiation

1. Less than or equal to 10 mrad/quarter.
2. Less than or equal to 20 mrad/year.

#### b.,c. For Iodine-131, for Iodine-133, and for all radionuclides in particulate form with half-lives greater than 8 days.

##### Dose Rate

1. Less than 1500 mrem/year

##### Dose

1. Less than or equal to 7.5 mrem/quarter
2. Less than or equal to 15 mrem/year.

#### d. For Liquid

Less than or equal to 3 mrem to the whole body during any calendar quarter.

Less than or equal to 10 mrem to any organ during any calendar quarter.

Less than or equal to 6 mrem to the whole body during any calendar year.

Less than or equal to 20 mrem to any organ during any calendar year.

## 2. Maximum Permissible Concentration

a., b., c., For fission and activation gases, iodines, and particulates with half-lives greater than 8 days, allowable release limits are calculated by solving equations 10.1 and 10.2 from the Offsite Dose Calculation Manual. The alarm setpoint is conservatively set at approximately 10% of the 10CFR20 limit.

d. For liquid effluents allowable release limits are calculated by solving equations 10.3 and 10.4 from the Offsite Dose Calculation manual. The MPC values used for the monitors were as follows:

radwaste discharge  $3.13\text{E}-06$  uCi/ml  
service water  $2.0\text{E}-05$  uCi/ml

## 3. Average Energy

The average gamma energy used to calculate the alarm setpoints for the noble gas monitors was 0.412 Mev for the First quarter, and 0.412 Mev for the Second quarter.

## 4. Measurements and Approximations of Total Radioactivity

- a. Fission and Activation Gases:
- b. Iodines:
- c. Particulates:

The main chimney and reactor building ventilation exhaust systems are continually monitored for iodines and particulates. These samples are pulled every 7 days and analyzed by gamma isotopic. The particulate papers are composited every 31 days and sent to a vendor for Sr 89-90 and gross alpha analysis. Noble gas grab samples are pulled and analyzed by gamma isotopic weekly. Tritium samples are pulled and analyzed every month.

The continuous strip chart recorders for the monitors on the release points are reviewed monthly for spikes and the activity released is calculated. An additional calculated activity for noble gases is added to the Main chimney release each month. This calculation is done because most of the grab samples show less than the lower limit of detection due to the low amount of activity and the large dilution flow at the sample point. The calculation takes into account the normal offgas train and the gland steam contribution to the release.

The average flow at the release points are used to calculate the curies released.

d. Liquid Effluents

The river discharge tanks are analyzed before discharge by gamma isotopic. A composite representative portion of this sample saved. This is composited with other discharges that occurred every 31 days and is analyzed for tritium and gross alpha. The batch composites are composited quarterly and sent to a vendor for Sr 89-90 and Fe 55. The discharge bay is sampled every 31 days and analyzed by gamma isotopic, for tritium and gross alpha. It is sampled quarterly and sent to a vendor for Sr 89-90 and Fe 55 analysis.

The tank volumes and activities are used to calculate the curies released for the River Discharge Tank. The total water released during the quarter and the activity is used to calculate the diluted activity released at the discharge bay, from batch discharges.

e. Estimated Total Error Percent

The estimated total error percents were calculated by taking the square root of the sum of the squares of errors for sampling and measurement parameters. The estimated total error percent for the solid waste radwaste curies is 12.3%.

f. Less than the lower limit of detection (<LLD).

Samples are analyzed such that the Technical Specification LLD requirements are met. When a nuclide is not detected during the quarter then <LLD is reported.

5. Batch Releases

a. Liquid

1. number of releases 24
2. total time 20,793 minutes
3. maximum time 953 minutes
4. average time 866 minutes
5. minimum time 710 minutes
6. average stream flow, discharge 58.7 gpm,  
dilution 1.61E+05 gpm.

b. Gaseous

NONE

## 6. Abnormal Releases

### a. Gaseous

NONE

### b. Liquid

On September 14, 1993, a leak was identified on the U-1 "B" RHR Heat Exchanger. Unit One was at full power at this time. The total activity released to the environment from this leak was  $1.03\text{E-}05$  Ci from 1/01/95 to 6/30/95, and was added to the monthly liquid release summary for each month. In this report, the activity is reported under the "Continuous Liquid Release" column.

This leak is scheduled for repair during early 1996.

On March 1, 1994, a leak was identified on the U-2 "A" RHR Heat Exchanger. Unit Two was at full power at this time. The total activity released to the environment from this leak was  $2.13\text{E-}05$  Ci from 1/01/95 to 3/5/95, and was added to the monthly liquid release summary for each month. In this report, the activity is reported under the "Continuous Liquid Release" column.

This leak was repaired during the refuel outage on U-2 in March 1995.