

LICENSEE EVENT REPORT

50-285/76-7

CONTROL BLOCK:

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: [01] N E F C S 1 [00] 00-000000-00 [4] 11111 [03] 03
7 8 9 14 15 25 26 30 31 32

CATEGORY: [01] CONT [57] [58] REPORT TYPE: [L] [59] REPORT SOURCE: [L] [60] DOCKET NUMBER: [05] 00-0285 [61] 032776 [68] 040876 [74] 040876 [75] 040876 [80] 040876

EVENT DESCRIPTION

[02] During unit operation at approximately 60 percent, Channel A APD positive limit was
7 8 9 80
[03] noted to be reading 12.1 which exceeded the limit of 4.60 at 60 percent power. The
7 8 9 80
[04] APD calculator is one of 4 redundant channels.
7 8 9 80
[05]
7 8 9 80
[06]
7 8 9 80

SYSTEM CODE: [07] I A [8] [9] CAUSE CODE: [E] [10] COMPONENT CODE: I N S T R U [11] N [12] PRIME COMPONENT SUPPLIER: B 1 6 5 [13] Y [14] VIOLATION: Y [15]
7 8 9 10 11 12 17 43 44 47 48

CAUSE DESCRIPTION

[08] The Bell and Howell 19-301A adder-subtractor module which is used to generate the
7 8 9 80
[09] upper limit was found to have gone into oscillation and saturated causing the positive
7 8 9 80
[10] limit to go out of specification. The module was replaced, the calculator was
7 8 9 80

FACILITY STATUS: [11] E [12] % POWER: 060 [13] OTHER STATUS: NA [14] METHOD OF DISCOVERY: B [15] DISCOVERY DESCRIPTION: Operator Checks
7 8 9 10 12 13 44 45 46 80

FORM OF ACTIVITY RELEASED: [12] Z [13] CONTENT OF RELEASE: Z [14] AMOUNT OF ACTIVITY: NA [15] LOCATION OF RELEASE: NA
7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES

[13] NUMBER: 000 [14] TYPE: Z [15] DESCRIPTION: NA
7 8 9 11 12 13 80

PERSONNEL INJURIES

[14] NUMBER: 000 [15] DESCRIPTION: NA
7 8 9 11 12 80

OFFSITE CONSEQUENCES

[15] NA
7 8 9 80

LOSS OR DAMAGE TO FACILITY

[16] TYPE: Z [17] DESCRIPTION: NA
7 8 9 10 80

PUBLICITY

[17] NA
7 8 9 80

8403260384 760408
PDR ADOCK 05000285
S PDR

ADDITIONAL FACTORS

[18] Description of Cause (continued): returned to service.
7 8 9 80

[19]
7 8 9 80

NAME: Robert Mehaffey

PHONE: 402-426-4011

ATTACHMENT 1

Safety Analysis

The failure of A channel APD occurred at 60 percent power with the unit at steady state operation. At that time channel B High Power, TMLP, and APD were in bypass as a result of the installation of a new linear power excore detector which required calibration to be done at a steady state power of 60 percent following a 3-day stabilization period. Channel B APD trip unit was immediately placed in trip by the operators. Channels C and D APD were operable. A one-out-of-two logic would have been required to trip the reactor is required. The requirement of Technical Specification Table 2-2 for two operable channels (the reactor must be reduced to 70 percent or less power and one of the inoperable channels be placed in trip) were met.

ATTACHMENT 2

Corrective Action

The Bell and Howell 19-301A Adder-Subtractor module used to generate the positive limit was replaced and Surveillance Test ST-RPS-12 Section F.2 was performed. Proper calculator operation was verified and the calculator returned to service.

The APD calculator limits are checked once per shift by Surveillance Test ST-RPS-12 Section F.1 and an operational check is made once per month by Surveillance Test ST-RPS-12 Section F.2.

ATTACHMENT 3

Failure Data

Related failures were reported as listed:

Abnormal Occurrence 50-285/75-12

Abnormal Occurrence 50-285/75-15

LER 50-285/76-6

LER 50-285/76-12



Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

April 13, 1976
FC-116-76

Mr. E. Morris Howard
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive
Suite 1000
Arlington, TX 76012



Dear Mr. Howard:

Reference: Fort Calhoun Station Unit No. 1
Docket No. 50-285

In accordance with the Fort Calhoun Station's Technical Specifications, the Omaha Public Power District, as holder of facility operating license DPR-40, submits three copies of the following licensee event report 50-285/76-7 to satisfy the requirements of Regulatory Guide 1.16.

Sincerely,

W. C. Jones
Section Manager
Operations

WCJ/WDD:rge

Enclosure

cc: Director, Office of Management
Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (3)

Director, Office of Inspection and
Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (30)

Mr. L. C. Shalla
SARC Chairman
PRC Chairman
Fort Calhoun File (2)

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