

# LICENSEE EVENT REPORT

50-285/76-10

CONTROL BLOCK:

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME 01 N E F C S 1														LICENSE NUMBER 00-000000-00														LICENSE TYPE 41111										EVENT TYPE 03													
CATEGORY 01 CONT														REPORT TYPE L				REPORT SOURCE L				DOCKET NUMBER 050-0285										EVENT DATE 040176										REPORT DATE 040876									

## EVENT DESCRIPTION

02 During operator checks of the Variable High Power Margin to Trip Meter it was noted  
 03 that B High Power Margin to Trip had drifted from 4.2 percent to 6 percent power. A  
 04 subsequent check of the high power trip point showed a high power trip would occur  
 05 at 106.2 percent power. A 1 percent calimetric error would have resulted in an  
 06 actual trip point of 107.2 percent which exceeds the limit of 106.5 percent. (con't)

SYSTEM CODE 07 I A				CAUSE CODE E		COMPONENT CODE I N S T R U										PRIME COMPONENT SUPPLIER N		COMPONENT MANUFACTURER B 1 6 5										VIOLATION Y	
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## CAUSE DESCRIPTION

08 The Bell and Howell 19-317-2X feed back limiter module used to generate the high  
 09 power trip setpoint was found to have drifted. The module was calibrated and op  
 10 operated satisfactorily.

FACILITY STATUS 11 E		% POWER 100				OTHER STATUS NA				METHOD OF DISCOVERY B		DISCOVERY DESCRIPTION Operator Checks									
FORM OF ACTIVITY RELEASED 12 Z		CONTENT OF RELEASE Z				AMOUNT OF ACTIVITY NA				LOCATION OF RELEASE											

## PERSONNEL EXPOSURES

NUMBER 13 000				TYPE Z		DESCRIPTION NA									
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## PERSONNEL INJURIES

NUMBER 14 000				DESCRIPTION NA									
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## OFFSITE CONSEQUENCES

15 NA

## LOSS OR DAMAGE TO FACILITY

TYPE 16 Z		DESCRIPTION NA									
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## PUBLICITY

17 NA

8403260324 760408  
PDR ADOCK 05000285  
S PDR

## ADDITIONAL FACTORS

18 Description of Event (continued): The high power trip is one of four redundant  
 19 channels.

NAME: Robert Mehaffey

PHONE: 402-426-4011

ATTACHMENT 1

Safety Analysis

The Reactor Protective System is so designed that the failure of B RPS High Power trip unit to trip at 104.4 percent power (which takes into account a 1.1 percent instrument error and a 1.0 percent calimetric error to insure a trip at the Technical Specification Limit of 106.5 percent) would not have prevented a reactor trip if the unit power had exceeded 106.5 percent power. Under normal operating conditions the reactor protective system is in a two-out-of-four logic configuration. The failure of B high power trip channel placed the RPS in a two-out-of-three logic. The two-out-of-three logic exceeds the minimum required for safe reactor operation as stated in Table 2-2 of the Technical Specifications. Channels A, C and D were operational.

ATTACHMENT 2

Corrective Action

The Bell and Howell 19-317-2X feed back limiter module was recalibrated. Surveillance Test ST-RPS-1 Section F.2 which checks the variable high power trip channels was done on B channel. Proper operation of B channel was verified and then returned to service.

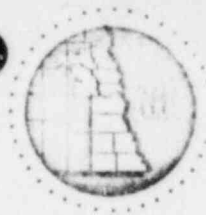
ATTACHMENT 3

Failure Data

The drift of the high power setpoint was the first failure of this type.

# Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402



April 28, 1976  
FC-149-76



Mr. E. Morris Howard  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive  
Suite 1000  
Arlington, TX 76012

Dear Mr. Howard:

Reference: Fort Calhoun Station Unit No. 1  
Docket No. 50-285

In accordance with the Fort Calhoun Station's Technical Specifications, the Omaha Public Power District, as holder of facility operating license DPR-40, submits three copies of the following amended licensee event report 50-285/76-10 to satisfy the requirements of Regulatory Guide 1.16.

Sincerely,

W. C. Jones  
Section Manager  
Operations

WCJ/WDD:rge

Enclosure

cc: Director, Office of Management  
Information and Program Control  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555 (3)

Director, Office of Inspection and  
Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555 (30)

Mr. L. C. Shalla  
SARC Chairman  
PRC Chairman  
Fort Calhoun File (2)

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