

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 8 5										PAGE (3) 1 OF 2					
TITLE (4) Unit Trip																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)											
0	3	0	9	8	4	8	4	0	0	6	0	0	0	3	2	0	8	4	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																							
N		20.402(b)					20.405(e)					<input checked="" type="checkbox"/> 50.73(a)(2)(iv)					73.71(b)								
POWER LEVEL (10)		20.406(a)(1)(i)					50.36(e)(1)					50.73(a)(2)(v)					73.71(c)								
0		9					20.406(a)(1)(ii)					50.36(e)(2)					<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 306A)								
16		20.406(a)(1)(iii)					50.73(a)(2)(i)					50.73(a)(2)(vi)(A)													
		20.406(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(vii)(B)													
		20.406(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																									
NAME												TELEPHONE NUMBER													
John J. Anderson												AREA CODE 9 1 4 7 3 9 - 8 2 0 0													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC															
x	J	C	7	2		W	1	2	0	Y															
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)				MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 9, 1984, with the reactor at 96 percent power, a turbine trip was initiated during monthly Reactor Protection System (RPS) testing. The cause for the turbine trip was found to be the inadvertant closure of a cell switch associated with Reactor Trip Bypass Breaker No. 52/BYB. Investigation determined that a misaligned catch plate in the breaker cubicle assembly led to the closure of the cell switch during testing. The catch was realigned and the breaker was tested and returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 0 6	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On March 9, 1984, at 0832 hours, a turbine trip and subsequent reactor trip were initiated during monthly Reactor Protection System (RPS) testing. Reactor power was constant at 96 percent prior to the trip.

Investigation determined that cycling the Train B Bypass Breaker (52/BYB) in accordance with the test procedure had led to an inadvertant closure of the breaker's cell switch. The purpose of the cell switch is to maintain the integrity of the trip logic matrices by automatically closing when the reactor trip breaker is removed from its cubicle. The inadvertant cell switch closure was found to be caused by a misaligned catch plate in the breaker cubicle assembly, which allowed a slight breaker misalignment within the cubicle.

As the test proceeded, the Train B Bypass Breaker was closed, and the Train B Trip Breaker (52/RTB) was opened. As Breaker 52/RTB opened, its associated "b" contacts closed as designed. The circuit to the automatic turbine trip devices was completed by the "b" contacts of Breaker 52/RTB and the cell switch of Breaker 52/BYB.

The catch was realigned, and Breaker 52/BYB (Westinghouse DB-50), was returned to service. Inspection of the three remaining RPS breakers revealed no similar problem.

All equipment associated with the reactor trip performed its designated function. The unit was synchronized to the bus on March 10, 1984 at 0520 hours.

No similar event has been reported in an LER to date.

This event is reportable under 10CFR50.73(a)(2)(iv) which became effective January 1, 1984.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200



March 20, 1984
IP-FWG-911

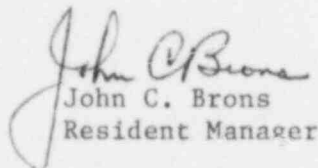
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 84-006-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,


John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
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