

LICEE (SEE EVENT REPORT (LER))

FACILITY NAME (1) Dresden Nuclear Power Station										DOCKET NUMBER (2) 0 5 0 0 0 2 1 3 1 7										PAGE (3) 1 OF 0 2																													
TITLE (4) Core Spray M0-2-1402-3A Failed to Close																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER(S)												
0 2			2 2			8 4			8 4			— 0 0 4			— 0 0			0 3			2 0			8 4			N/A										0 5 0 0 0 0												
0 2			2 2			8 4			8 4			— 0 0 4			— 0 0			0 3			2 0			8 4			N/A										0 5 0 0 0 0												
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																							
POWER LEVEL (10) 0 5 0										20.402(b)										20.405(e)										50.73(a)(2)(iv)										73.71(b)									
										20.405(a)(1)(i)										50.38(a)(1)										50.73(a)(2)(v)										73.71(e)									
										20.405(a)(1)(ii)										50.38(a)(2)										50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
										20.405(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(viii)(A)																			
										20.405(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																			
										20.405(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME Lawrence Coyle (X526)																				TELEPHONE NUMBER 8 1 5 9 4 2 - 2 9 2 0																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NPDs			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NPDs																						
X			B M			C N T R			G 0 8 0			N																																					
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
X YES (If yes, complete EXPECTED SUBMISSION DATE)																				NO										EXPECTED SUBMISSION DATE (15)																			
																														0 6 0 1 8 4																			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing DOS 1400-2, Core Spray Valve Operability Test, Core Spray suction valve M0-2-1402-3A failed to close from the control room. Safety significance was minimal since the 'B' Core Spray loop and all other ECCS systems were operable. Previous occurrence reported by R.O. 83-69/03L-0 on Docket #050-237.

The cause was a faulty auxiliary breaker contact (G.E. CR105X) in the valve control circuitry. Both auxiliary contacts on the breaker were replaced and DOS 1400-2 was performed to verify valve operability. Previous to this event, similar auxiliary contacts had been sent to the Operational Analysis Department for analysis. Also General Electric has been requested to review causes of contact problems. A supplemental report will be submitted when both analyses are completed.

8403260223 840320
PDR ADOCK 05000237
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 9/31/85

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 0 4	— 0 0 0	2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

While performing DOS 1400-2, Core Spray Valve Operability Test, Core Spray suction valve M0-2-1402-3A failed to close from the control room. Safety significance was minimal since the redundant Core Spray 'B' loop was operable and capable of providing core cooling. Previous to this event, the Core Spray Injection 2-1402-25A failed to operate from the control room in conjunction with the Unit 2/3 diesel fuel storage tank out of service for an inspection. A GSEP unusual event was declared and a normal unit shutdown was initiated. A previous similar occurrence was reported by R.O. 83-69/03L-0 on Docket #050-237.

The cause of the event was a faulty auxiliary breaker contact (G.E. CR105X) in the valve control circuitry. Both auxiliary contacts on the breaker were replaced. Prior to this, the 2-1402-25A had been returned to service. The 2-1402-3A was returned to service and the Core Spray 'A' loop was declared operable. The unit shutdown was terminated and the unit began increasing load. Similar faulty auxiliary contacts have been previously sent to the Operational Analysis Department for analysis. Also, General Electric has been requested to review possible causes of contact problems. A supplemental report will be submitted when both analyses are completed.



DEVIATION REPORT

Commonwealth Edison

DVR NO.

12 - 2 - 84 - 20
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

U 2 MO 1402-3A Valve

OCCURRED

2/22/84

0913

DATE

TIME

SYSTEM AFFECTED 1400

PLANT STATUS AT TIME OF EVENT

Core Spray

MODE Run, POWER(%) 50

33721

TESTING

WORK REQUEST NO.

☒

YES

☐

NO

DESCRIPTION OF EVENT

Valve failed to close when operated during DOS 1400-2. Breaker was racked out, the main contactor was manually exercised and the breaker was then racked back in. The valve was then cycled three times. A work request was written to inspect the breaker.

POTENTIALLY SIGNIFICANT EVENT PER HSD DIRECTIVE A-07

☐ YES☒ NO10CFR50.72 NRC RED PHONE
NOTIFICATION MADE☒ 1 HOUR☐ 4 HOUR

1010

☐ NO

TIME

RESPONSIBLE SUPERVISOR J. Gates

DATE 2/22/84

PART 3 OPERATING ENGINEER'S COMMENTS

Corrective maintenance was performed on the breaker by the Electrical Maintenance department, and the MO-2-1402-3A valve was declared operable. During the time that the MO-2-1402-3A valve was inoperable, the U-2/3 diesel generator was out-of-service for an annual inspection. Unit 2 had previously started a normal unit shutdown and an unusual event had been declared, due to the MO-2-1402-25A valve being inoperable. (over)

☐ NON REPORTABLE EVENT☒ 30 DAY REPORTABLE/
10CFR 50.73(a)(2)(i)☐ 5 DAY REPORT PER 10CFR21☐ ANNUAL/SPECIAL REPORT REQUIRED
PER

A.I.R. #

L.E.R. # 84-004-0

NOTIFICATION N/A

REGION III

DATE

TIME

TELECOPY

Dennis P. Galle

2-27-84

1130

NSD

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE
IF ABOVE NOTIFICATION IS PER 10CFR21

TELEPHONE N/A

CECO CORPORATE OFFICER

DATE

TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

John M. Almer

2/24/84

OPERATING ENGINEER

DATE

INVESTIGATION REPORT & RESOLUTION
ACCEPTED BY STATION REVIEWRESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTION

(Form 15-52-1)

1-84

STATION SUPERINTENDENT

DATE

OPERATING ENGINEER'S COMMENTS

When the M0-2-1402-3A valve was declared operable, the unusual event was terminated.



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

March 20, 1984

DJS Ltr. #84-263

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #84-004-0, Docket # 050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(i).

D. J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/jnt

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

LE22
1/1