

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 5 6				PAGE (3) 1 OF 0 4											
TITLE (4) Safety relief valve failed to lift when tested																									
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	2	2	0	8	4	8	4	0	0	2	0	0	0	3	2	0	8	4	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																							
5		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)											
0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME										TELEPHONE NUMBER															
Steven B. Tipps, Superintendent of Regulatory Compliance										AREA CODE		9 1 2 3 6 7 + 1 7 8 5 1													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS															
X	S	R																							
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of Wyle Laboratories testing of main steam safety relief valves (SRVs), five SRVs failed to lift in the 1% tolerance range required by Tech. Specs. Section 3.4.2.1.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)					
		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER								
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6 8 4	—	0	0	2	—	0	0	0	2	OF	0	4

TEXT (If more space is required, use additional NRC Form 386A's) (17)

REQUIREMENT FOR REPORT:

This 30-day report is required by 10 CFR 50.73 (a)(2)(i) due to this event's showing that the plant was not in compliance with Tech. Specs. section 3.4.2.1.

PLANT CONDITIONS AT THE TIME OF THE EVENT(S):

The plant was in a refueling outage when the main steam relief valves were removed and shipped to Wyle Laboratories for testing.

DETAILED DESCRIPTION OF THE EVENT(S):

With Unit 2 in refueling mode, bench testing was in progress per the "MAIN STEAM RELIEF VALVES MAINTENANCE" procedure (HNP-2-6020) by Wyle Laboratories. Wyle personnel found 5 main steam relief valves (SRVs) out of Tech. Specs. section 3.4.2.1. acceptable limits as listed below:

SRVs LISTED BY MASTER PARTS LIST NUMBERS	EXPECTED LIFT SETPOINT IN PSID	TECH. SPECS. SECTION 3.4.2.1 TOLERANCE	AS FOUND LIFT PRESSURE IN PSID	% ABOVE SET-POINT
2B21-F013H	1110	+ 1%	would not lift	
2B21-F013D	1100	+ 1%	1114	1.3
2B21-F013F	1090	+ 1%	1105	1.4
2B21-F013K	1100	+ 1%	1122	2.0
2B21-F013L	1110	+ 1%	1133	2.0

SUMMARY ASSESSMENT OF ACTUAL AND POTENTIAL SAFETY CONSEQUENCES AND IMPLICATIONS:

There were no actual or potential safety consequences during plant operation since the six remaining SRVs remained operable. Additionally, the four SRVs which only had a slight set-point problem would have functioned also. In summary, only one of the SRVs (i.e., 2B21-F013H) would not have functioned.

STATUS OF REDUNDANT OR BACKUP SUBSYSTEMS AND/OR SYSTEMS:

Three SRVs with expected lift settings of 1090 PSID were found in tolerance. Also, two SRVs with expected lift settings of 1100 PSID and one SRV with an expected lift setting of 1110 PSID were found in tolerance.

JUSTIFICATION FOR CONTINUED OPERATION:

SRVs are not required in the refuel mode. All SRVs will be corrected and reinstalled prior to Unit 2 start-up.

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EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6 8 4 - 0 0 2 - 0 0 0 3				OF	0	4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

IF REPETITIVE:NUMBER OF PREVIOUS LER:

This is a repetitive event, for SRVs not actuating within their Tech. Specs. requirement, as last reported on LER 50-366/1983-030.

WHY CORRECTIVE ACTION DID NOT PREVENT RECURRENCE:

The Target Rock model 7567F safety relief valves used on Units 1 and 2 are only certified to a  $\pm 3\%$  tolerance by Target Rock.

IMPACT TO OTHER SYSTEMS AND/OR UNIT:

Target Rock Model 7567F safety relief valves are utilized by both Unit 1 and Unit 2 as main steam safety relief valves.

CAUSE(S) OF THE EVENTS(S):

The cause of 2B21-F013H failure to lift was due to corrosion build up on its pilot assembly. The cause of the other valves' being outside of the  $\pm 1\%$  Tech. Specs. tolerance is a setpoint that is too conservative for these SRVs to consistently meet.

IMMEDIATE CORRECTIVE ACTION(S):

The pilot assembly for all tested SRVs were refurbished (i.e., cleaned, tolerances checked, and remachined as necessary). All SRVs were calibrated per the requirement of Tech. Specs.

SUPPLEMENTAL CORRECTIVE ACTION(S):

No supplemental corrective action is required.

SCHEDULED (FUTURE) CORRECTIVE ACTION(S):

No scheduled (future) corrective action is required.

ACTION(S) TO PREVENT RECURRENCE (IF DIFFERENT FROM CORRECTIVE ACTIONS):

General Electric is investigating whether improving our water treatment or changing the pilot assembly material would correct the corrosion problem found on 2B21-F013H.

Additionally, an investigation is being initiated as to the feasibility of initiating a Document Change Request to change the Tech. Specs. tolerance to a value that the valves can meet.

TEXT (If more space is required, use additional NRC Form 305A's) (17)

### IDENTIFICATION OF EACH FAILED COMPONENT

MASTER PARTS LIST NUMBER.	MANUFACTURER	MODEL NUMBER
2B21-F013H	Target Rock	7567F
2B21-F013D	Target Rock	7567F
2B21-F013F	Target Rock	7567F
2B21-F013K	Target Rock	7567F
2B21-F013L	Target Rock	7567F

Georgia Power Company  
Post Office Box 439  
Baxley, Georgia 31513  
Telephone 912 367-7781  
912 537-9444



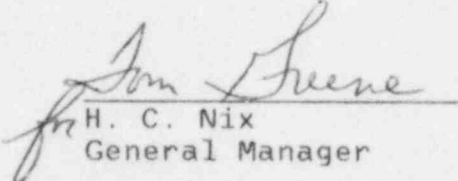
Edwin I. Hatch Nuclear Plant

March 20, 1984  
GM-84-249

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-02. This report is required by 10 CFR50.73(A)(2)(I)(B).

  
H. C. Nix  
General Manager

HCN/SBT/vlt

xc: R. J. Kelly  
R. E. Conway  
J. T. Beckham, Jr.  
P. D. Rice  
K. M. Gillespie  
S. B. Tipps  
R. D. Baker  
Control Room  
Document Control

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