

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 8 6				PAGE (3) 1 OF 012		
TITLE (4) Unit Trip																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)				
02	20	84	84	005	0	03	20	84				0 5 0 0 0				
OPERATING MODE (9) N			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)													
POWER LEVEL (10) 0.1910			20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)	
			20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)	
			20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
			20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)																
NAME John J. Anderson										TELEPHONE NUMBER AREA CODE 914 739-1820						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On February 20, 1984, with the reactor at 90 percent power, a reactor trip was initiated on a steam flow/feed flow mismatch on No. 32 steam generator. The mismatch was caused by closure of main feed regulating valve as a result of the failure of a trip solenoid. Subsequent investigation indicated that water leakage into the solenoid's terminal box was responsible for the solenoid failure. The leads were replaced, the terminal box was drained and sealed, and the unit returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 8 4 - 0 0 5 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT: If more space is required, use additional NRC Form 365A's (17)

At 1945 hours on February 20, 1984, a reactor trip was initiated on a steam flow/feed flow mismatch on No. 32 steam generator. Reactor power was constant at 90 percent prior to the trip. The immediate cause for the mismatch was an inadvertant closure of valve No. FCV-427, which is the main feed regulating valve for No. 32 steam generator.

Upon investigation, it was determined that water had collected in the terminal box for the trip solenoid valve associated with Flow Control Valve FCV-427. This water caused the solenoid's leads to corrode and subsequently fail. The solenoid valve became de-energized, terminating the control air signal to FCV-427. The main feed regulating valve closed, as designed, on a loss of control air. It is believed that water leaked into the terminal box as a result of various component leakages over a period of time. No leak was present at the time of the trip. The necessary leads were repaired and the terminal strips were replaced. The water was removed, and all equipment returned to normal operation.

All terminal boxes associated with the feedwater regulating valves were opened and inspected for water. To prevent recurrences, a sealant was placed at the points where cables penetrate each terminal box casing, and drain holes were drilled in the bottom of these casings.

All equipment associated with the trip performed its designated function. The reactor was returned to criticality at 2257 hours in accordance with established procedures.

No similar event has been reported in an LER to date.

This event is reportable under 10CFR50.73(a)(2)(iv), which became effective January 1, 1984.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200



March 20, 1984
IP-FWG-910

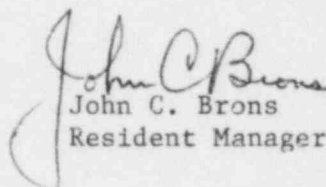
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 84-005-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,


John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
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