



ENTERGY

Entergy Operations, Inc.

1448 S.R. 333

Russellville, AR 72801

Tel 501 858-5000

August 15, 1995

1CAN089504

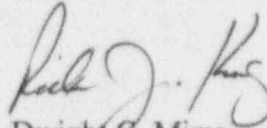
U. S. Nuclear Regulatory Commission
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Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 1 Monthly Operating Report for July 1995 is attached. This report is submitted in accordance with ANO-1 Technical Specification 6.12.2.3.

Very truly yours,


for Dwight C. Mims
Director, Licensing

DCM/eas

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U. S. NRC
August 15, 1995
1CAN089504

cc: Mr. Leonard J. Callan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One
P.O. Box 310
London, AR 72847

Mr. George Kalman
NRR Project Manager Region IV/ANO-1 & 2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

OPERATING DATA REPORT

DOCKET NO: 50-313
 DATE: August 15, 1995
 COMPLETED BY: M. S. Whitt
 TELEPHONE: (501) 858-5560

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 1
2. Reporting Period: July 1-31
3. Licensed Thermal Power (MWt): 2,568
4. Nameplate Rating (Gross MWe): 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted. If Any (Net MWe): _____
10. Reasons For Restrictions. If Any: _____

	<u>MONTH</u>	<u>YR-TO-DATE</u>	<u>CUMULATIVE</u>
11. Hours in Reporting Period	744.0	5,087.0	180,714.0
12. Number of Hours Reactor was Critical	716.9	3,902.8	133,159.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,044.0
14. Hours Generator On-Line	707.1	3,821.0	130,808.9
15. Unit Reserve Shutdown Hours	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH)	1,802,343	8,911,267	303,065,156
17. Gross Electrical Energy Generated (MWH)	609,463	3,034,153	101,346,073
18. Net Electrical Energy Generated (MWH)	581,465	2,884,690	96,410,179
19. Unit Service Factor	95.0	75.1	72.4
20. Unit Availability Factor	95.0	75.1	72.8
21. Unit Capacity Factor (Using MDC Net)	93.5	67.8	63.8
22. Unit Capacity Factor (Using DER Net)	91.9	66.7	62.8
23. Unit Forced Outage Rate	5.0	3.6	10.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End of Report Period. Estimated Date of Startup: _____
26. Units in Test Status (Prior to Commercial Operation): _____

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	<u>08/06/74</u>
INITIAL ELECTRICITY	_____	<u>08/17/74</u>
COMMERCIAL OPERATION	_____	<u>12/19/74</u>

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313
UNIT: One
DATE: August 15, 1995
COMPLETED BY: M. S. Whitt
TELEPHONE: (501) 858-5560

MONTH July 1995

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	836
2	838
3	837
4	837
5	839
6	839
7	837
8	837
9	835
10	833
11	830
12	819
13	826
14	827
15	262
16	-16
17	715
18	827
19	828
20	828
21	830
22	829
23	830
24	831
25	831
26	830
27	828
28	825
29	825
30	826
31	827

AVGS: 782

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

OPERATING SUMMARY

JULY 1995

UNIT ONE

The unit began the month of July at 100% power.

At 0250 hours on the twelfth, a power reduction to 99% was required to prevent procedural limits on condenser vacuum from being exceeded due to high lake temperatures. After determining that the low condenser vacuum limit of 26.5 in HgA was incorrect, the proper limit was applied and the unit returned to 100% power at 2052 hours the same day. While performing repair work to generator output breaker B5118, the unit experienced an automatic reactor / turbine trip at 0818 hours on the fifteenth. The trip was the result of the electro-hydraulic control system incorrectly sensing that both generator output breakers (B5118 and B5114) were open due to a contact problem with breaker B5114. While modifications were being performed on the breakers, the unit attained criticality at 1126 hours on the sixteenth. Upon completion of repairs and modifications to the breakers, the unit was placed on-line at 2112 hours on the sixteenth. The unit performed normal startup procedures and attained 100% power at 0826 hours on the seventeenth.

The unit operated the remainder of the month at 100% power.

UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR JULY 1995

DOCKET NO.	50-313
UNIT NAME	ANO Unit 1
DATE	August 15, 1995
COMPLETED BY	M. S. Whitt
TELEPHONE	501-858-5560

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSEE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
95-08	950715	F	36.9	A	3	1-95-009-00	FK	BKR	When generator output breaker B5118 was opened so repair work could be performed, the parallel breaker B5114 had a bad contact that resulted in the EHC system sensing that both breakers were open. This resulted in the main turbine governor and intercept valves receiving a close signal which initiated the reactor trip and generator / turbine lockout. The root cause of the contact failure is indeterminate. A failure analysis will be conducted after the contact switch is repaired during the next refueling outage. Spare status contacts were wired in parallel with all of the main generator output circuit breaker status contacts in the EHC logic circuit to prevent a single contact failure from giving a false breaker position indication.

1
F: Forced
S: Scheduled

2
Reason:
A - Equipment Failure (Explain)
B - Maintenance of Test
C - Refueling
D- Regulatory Restriction
E - Operator Training & License Examination
F - Administration
G - Operational Error
H - Other (Explain)

3
Method:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Continuation
5 - Load Reduction
9 - Other

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 1
2. Scheduled date for next refueling shutdown: September 20, 1996
3. Scheduled date for restart following refueling: November 4, 1996
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10CFR Section 50.59)?

No, No
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

NA
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None planned
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

a) 177 b) 745
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

present 968 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

DATE: 1996 (Loss of full core off-load capability)