

LICENSEE EVENT REPORT

50-285/76-29

CONTROL BLOCK:

1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER										LICENSE TYPE					EVENT TYPE								
01	N	E	F	C	S	1								0	0	-	0	0	0	0	0	-	0	0			4	1	1	1	1		0	3			
7	8	9				14								15													25						30			31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER										EVENT DATE					REPORT DATE																
01	CON'T	L	L	0	5	0	-	0	2	8	5							0	8	3	1	7	6							0	9	2	3	7	6
7	8			57	58	59	60	61									68	69											74						80

EVENT DESCRIPTION

02 During power ascension at 1001 on August 31, 1976, reactor coolant was sampled and

03 analyses indicated iodine-131 equivalent 2.075 microcuries per gram. Resample at

04 1322 on the same day indicated iodine-131 equivalent decreased to 1.632. This value

05 was well within the numerical limit of Technical Specification 2.1.3.

06

SYSTEM CODE	CAUSE CODE	COMPONENT CODE						PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION	
07	C	B	E	Z	Z	Z	Z	Z	Z	Z	9	9	9	N
7	8	9	10	11	12				17	43	44		47	48

CAUSE DESCRIPTION

08 The iodine-131 equivalent increased as a result of a power transient which occurred

09 12 hours earlier. The numerical limit was exceeded, but all conditions of Technical

10 Specification 2.1.3 were fulfilled.

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION			
11	C	0	6	7	NA	B	Radiochemical Analysis
7	8	9	10	11	12	13	

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z	NA	NA
7	8	9	10

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION			
13	0	0	0	Z	NA
7	8	9	11	12	13

PERSONNEL INJURIES

NUMBER	DESCRIPTION			
14	0	0	0	NA
7	8	9	11	12

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION		
16	Z	NA	
7	8	9	10

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 See Attachments 1, 2 and 3

19

8403230168 760923
PDR ADOCK 05000285
S PDR

NAME: Fred F. Franco

PHONE: 402-426-4011

Safety Analysis

Maximum iodine concentrations are based on assumptions which include 0.53% fuel defects, steam generator tube ruptures and steam line breaks. None of these conditions were remotely present during the short period the iodine-131 equivalent exceeded 2.0 microcuries per gram. In addition, temporary increases in iodine concentrations are normal following power transients and continued operation permitted within safety considerations provided the activity trend indicates a decrease as was evident during this event.

Corrective Action

In conformance with Technical Specification 2.3.1 the cause was evaluated as an indication of approximately 0.10% fuel defects and the determination made that the iodine radioactivity was returning to a level within the specification. One basis for this determination was the identification of higher iodine-131 equivalent values experienced during a hot shutdown of the reactor on August 15, 1976. On that occasion the iodine-131 equivalent increased to approximately 18 microcuries per gram and the reactor remained in the hot shutdown condition for an additional 23 hours. During this time the decreasing trend for iodine-131 equivalent was established and an acceptable concentration verified prior to establishing the reactor critical and increasing the average coolant temperature above 532°F. As predicted, the iodine-131 equivalent had decreased much below the numerical limit within 3 hours on August 31, 1976. During the 3 hour period the reactor power level was not increased.

Failure Data

This is the first occurrence of the iodine-131 equivalent concentration being greater than 2 microcuries per gram when the reactor was critical.



Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

September 23, 1976

FC-286-76



Mr. E. Morris Howard
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive
Suite 1000
Arlington, TX 76012

Dear Mr. Howard:

Reference: Fort Calhoun Station Unit No. 1
Docket No. 50-285

In accordance with the Fort Calhoun Station's Technical Specifications, the Omaha Public Power District, as holder of facility operating license DPR-40, submits three copies of the following licensee event report 50-285/76-29 to satisfy the requirements of Regulatory Guide 1.16.

Sincerely,

W. C. Jones
Section Manager
Operations

WCJ/WDD:rge

Enclosures

cc: Director, Office of Management
Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (3)

Director, Office of Inspection and
Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (30)

Mr. L. C. Shalla
SARC Chairman
PRC Chairman
Fort Calhoun File (2)

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