

LICENSEE EVENT REPORT

50-285/76-30

CONTROL BLOCK: 1 2 3 4 5 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01 N E F C S 1										00-000000-00										41111					01	
7 8 9 14 15 25 26 30 31 32																										
CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER										EVENT DATE					REPORT DATE							
01 CONT		T	L	050-0285										090176					090276							
7 8		57	58	59 60 61 68 69 74 75 80																						

EVENT DESCRIPTION

02	During the performance of ST-RM-2 Section F.2, Process Monitor Checks, it was discover-	80
03	ed that the high alarm (control function) set point on Steam Generator Blowdown Moni-	80
04	tors RM-054A and RM-054B was set at 600,000 counts per minute which is higher than the	80
05	proper set points of 6,000 counts per minute for RM-054A and 4,000 counts per minute	80
06	for RM-054B. The setpoints were reset to the proper level. (LER 50-285/76-30)	80
7 8 9		80

SYSTEM CODE	CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VIOLATION
07 M C	A	I N S T R U					A	V 1 1 5			Y
7 8 9 10	11	12 17					43	44 47			48

CAUSE DESCRIPTION

08	During the performance of the monthly surveillance test ST-RM-2 Section F.2 on August	80
09	4, 1976, an unofficial copy of the process monitor setpoints was used as part of the	80
10	test. On this unofficial copy the setpoints of RM-054A and RM-054B were in error.	80
7 8 9		80

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11 E	067	NA	B	Monthly Surveillance Test
7 8 9	10 12 13		44 45 46	80
FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE	
12 Z	Z	NA	NA	
7 8 9	10 11	44 45	80	

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 000	Z	NA
7 8 9 11	12	13 80

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 000	NA
7 8 9 11	12 80

OFFSITE CONSEQUENCES

15 NA
7 8 9 80

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7 8 9 10	80

PUBLICITY

17 NA
7 8 9 80

ADDITIONAL FACTORS

18	See Attachments No. 1, 2 and 3.	80
7 8 9		80

19		80
7 8 9		80

NAME: J. M. Nagl/R. F. Mehaffey

PHONE: 402-426-4011

Safety Analysis

The improper setpoints in RM-054A and RM-054B High alarm, which provides Steam Generator Blowdown isolation, would have prevented the automation isolation of blowdown if required. However, blowdown monitor alert setpoints were found within specification and in addition, RM-056B monitors the blowdown as it is released into the river. If high steam generator activity were present, adequate indication and control was functional to insure that any potential radioactive release would not have occurred.

A review of the radio-chemistry analysis of the steam generator blowdown and a review of the continuous record of RM-054A and RM-054B show that at the time the radiation monitor isolation function was out of specification there was no activity which would have required the automatic isolation functions to be initiated.

Corrective Action

To prevent a reoccurrence of this incident in which improper setpoints were set into a safety system by the use of unofficial documents several steps have been taken:

- (1) The unofficial document which resulted in the error has been removed from the control room.
- (2) The surveillance test ST-RM-2 Section F.2 which requires the movement of the setpoints in its operational checks will be changed to insure that the Process Radiation Monitors have been returned to proper operation and proper setpoints.
- (3) An addition, as part of OPPD's personnel training, insuring that the documents used in performing all tests are the latest approved will be explained and re-emphasized.

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ATTACHMENT NO. 3

Failure Data

No component failure was involved.



FILE COPY

Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

September 3, 1976
FC-264-76



Mr. E. Morris Howard
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive
Suite 1000
Arlington, TX 76012

Dear Mr. Howard:

Reference: Fort Calhoun Station Unit No. 1
Docket No. 50-285

In accordance with the Fort Calhoun Station's Technical Specifications, the Omaha Public Power District, as holder of facility operating license DPR-40, submits three copies of the following licensee event report 50-285/76-30 to satisfy the requirements of Regulatory Guide 1.16.

Sincerely,

W. C. Jones
Section Manager
Operations

WCJ/WDD:rge

Enclosure

cc: Director, Office of Management
Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (3)

Director, Office of Inspection and
Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (30)

Mr. L. C. Shalla
SARC Chairman
PRC Chairman
Fort Calhoun File (2)

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