

Exhibit B

Monticello Nuclear Generating Plant

License Amendment Request Dated August 15, 1995

Technical Specification Pages Marked Up
with Proposed Wording Changes

Exhibit B consists of the existing Technical Specification pages marked up with the proposed changes. Existing pages affected by this change are listed below:

Page

104
158
159
160
161

3.0 LIMITING CONDITION FOR OPERATION

C. Containment Spray/Cooling System

1. Except as specified in 3.5.C.2, 3 and 4 below, both Containment Spray/Cooling Subsystems shall be operable whenever irradiated fuel is in the reactor vessel and reactor water temperature is greater than 212°F. A containment/spray cooling subsystem consists of the following equipment powered from one division:

2 RHR Service Water Pumps

1 Heat Exchanger

2 RHR Pumps*

Valves and piping necessary for:

Torus Cooling

Drywell Spray

2. One RHR Service Water Pump may be inoperable for 30 days.
3. One RHR Service Water Pump in each subsystem may be inoperable for 7 days.
4. One Containment Spray/Cooling Subsystem may be inoperable for 7 days.
5. If the requirements of 3.5.C.1, 2, 3 and 4 cannot be met, an orderly shutdown of the reactor will be initiated and the reactor water temperature shall be reduced to less than 212°F within 24 hours.

* For allowed out of service times for the RHR pumps see Section 3.5.A.

3.5/4.5

4.0 SURVEILLANCE REQUIREMENTS

C. Containment Spray/Cooling System

1. Demonstrate the RHR Service Water pumps develop 3,500 gpm flow rate against a 500 ft head when tested pursuant to Specification 4.15.B.
2. Test the valves in accordance with Specification 4.15.B.
3. Demonstrate the operability of the drywell spray headers and nozzles with an air test during each 5 year period.

10

3.0 LIMITING CONDITIONS FOR OPERATION

2. Primary Containment Integrity

- a. Primary Containment Integrity, as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel, except when performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 MW(t). Without Primary Containment Integrity, restore Primary Containment Integrity within one hour or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

2. Primary Containment Integrity

- a. Primary Containment Integrity shall be demonstrated after each closing of each penetration subject to Type B testing, if opened following a Type A or Type B test, by leak rate testing the seal with gas at \geq Pa, 42 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Surveillance Requirement 4.7.A.2.b, for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.6La.

Combined maximum flow path leakage

3.0 LIMITING CONDITIONS FOR OPERATION

- b. When Primary Containment Integrity is required, leakage rates shall be limited to:
1. An overall integrated leakage rate of less than or equal to L_a , 1.2 percent by weight of the containment air per 24 hours at Pa, 42 psig.
 2. A combined leakage rate of less than or equal to $0.6L_a$ for all penetrations and valves, ~~except for main steam isolation valves~~, subject to Type B and C tests when pressurized to Pa, 42 psig.
 3. Less than or equal to ~~11.5~~⁴⁶ scf per hour for all ~~any one~~ main steam isolation valves when tested at 25 psig.

With the measured overall integrated primary containment leakage rate exceeding $0.75L_a$, or the measured combined leakage rate for all penetrations and valves, ~~except main steam isolation valves~~, subject to Type B and C testing exceeding $0.6L_a$, or the measured leak rate exceeding ~~11.5~~⁴⁶ scf per hour for ~~any one~~ main steam isolation valves, restore leakage rates to less than or equal to these values prior to increasing reactor coolant system temperature above 212°F or, alternatively, restore measured leakage rates to within these limits within one hour or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

Combined maximum flow path

3.7/4.7

4.0 SURVEILLANCE REQUIREMENTS

- b. The primary containment leakage rates shall be demonstrated ~~at the following~~ in accordance with the test schedule ~~and shall be determined in conformance with the criteria, methods and provisions of 10 CFR Part 50; Appendix J as modified by approved exemptions.~~

- ~~Deleted~~
1. ~~Three Type A overall integrated containment leakage rate tests shall be conducted at 40 ± 10 month intervals* during shutdown at Pa during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.~~

- ~~Deleted~~
2. ~~If any periodic Type A test fails to meet $0.75L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $0.75L_a$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $0.75L_a$, at which time the above test schedule may be resumed.~~

- ~~Deleted~~
3. ~~All Type A test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.~~

~~*The second test of the second 10-year service period may be conducted during the 1989 refueling outage.~~

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

~~Deleted~~

4. ~~The accuracy of each Type A test shall be verified by a supplemental test which:~~
- ~~a. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25L_a$, and~~
 - ~~b. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test, and~~
 - ~~c. Requires the rate of gas injected into the containment or bled from the containment during the supplemental test to be limited between 75 to $125L_a$ of L_a .~~
5. ~~Type B and C tests shall be conducted with gas at $\geq P_a$ at each refueling shutdown (maximum interval of 24 months), except for tests involving the main steam line isolation valves. Main steam isolation valve tests shall be conducted with gas at > 25 psig each 18 months. A combined leakage rate of $< 0.6L_a$ shall be demonstrated for all penetrations and valves, except for main steam line isolation valves, subject to Type B and C tests. A leakage rate of < 11.5 scf per hour shall be demonstrated for each main steam line isolation valves.~~ all

Combined maximum flow path

3.0 LIMITING CONDITIONS FOR OPERATION

- c. When Primary Containment Integrity is required, the primary containment airlock shall be operable with:
1. Both doors closed except when the airlock is being used, then at least one airlock door shall be closed, and
 2. An overall airlock leakage rate of less than or equal to 0.05La at Pa or 0.007La at 10 psig.

With the primary containment airlock inoperable, maintain at least one airlock door closed and restore the airlock to Operable status within 24 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

By performing overall airlock leakage test in accordance with the schedule and criterion of 10CFR Part 50, Appendix J, as modified by approved exemptions.

4.0 SURVEILLANCE REQUIREMENTS

- c. The primary containment airlock shall be demonstrated operable:
1. ~~At each refueling shutdown, and at six month intervals thereafter, by conducting an overall airlock leakage test at $\geq P_a$ and demonstrating that overall airlock leakage rate is $\leq 0.05 L_a$. This test interval may be extended up to the next refueling outage (up to a maximum interval between tests at P_a of 24 months) if there have been no air lock openings since the last successful test at P_a .~~
Deleted
 2. ~~After each opening by conducting an overall airlock leakage test at ≥ 10 psig and verifying the leakage rate is $\leq 0.007 L_a$. If the airlock is being used for multiple openings, this test is not required after each opening, but shall be performed at least once per 72 hours.~~
 3. At six month intervals by verifying that only one door can be opened at a time. If the airlock has not been used since the last door interlock test, this test is not required.
- d. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.

Exhibit C

Monticello Nuclear Generating Plant

License Amendment Request Dated August 15, 1995

Revised Monticello Technical Specification Pages

Exhibit C consists of revised Technical Specification pages that incorporate the proposed changes. The pages included in this exhibit are as listed below. Note that due to removal of specific material and the resulting reconciliation, page 161 is no longer included.

Page

104
158
159
160

3.0 LIMITING CONDITION FOR OPERATION

C. Containment Spray/Cooling System

1. Except as specified in 3.5.C.2, 3 and 4 below, both Containment Spray/Cooling Subsystems shall be operable whenever irradiated fuel is in the reactor vessel and reactor water temperature is greater than 212°F. A containment/spray cooling subsystem consists of the following equipment powered from one division:

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1 Heat Exchanger

2 RHR Pumps*

Valves and piping necessary for:

Torus Cooling

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2. One RHR Service Water Pump may be inoperable for 30 days.
3. One RHR Service Water Pump in each subsystem may be inoperable for 7 days.
4. One Containment Spray/Cooling Subsystem may be inoperable for 7 days.
5. If the requirements of 3.5.C.1, 2, 3 and 4 cannot be met, an orderly shutdown of the reactor will be initiated and the reactor water temperature shall be reduced to less than 212°F within 24 hours.

* For allowed out of service times for the RHR pumps see Section 3.5.A.

4.0 SURVEILLANCE REQUIREMENTS

C. Containment Spray/Cooling System

1. Demonstrate the RHR Service Water pumps develop 3,500 gpm flow rate against a 500 ft head when tested pursuant to Specification 4.15.B.
2. Test the valves in accordance with Specification 4.15.B.
3. Demonstrate the operability of the drywell spray headers and nozzles with an air test during each 10 year period.

3.0 LIMITING CONDITIONS FOR OPERATION

2. Primary Containment Integrity

- a. Primary Containment Integrity as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel, except when performing low power physics test at atmospheric pressure during or after refueling at power levels not to exceed 5 MW(t). Without Primary Containment Integrity, restore Primary Containment Integrity within one hour or be in at least Hot Shutdown within the next 12 hours and Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

2. Primary Containment Integrity

- a. Primary Containment Integrity shall be demonstrated after each closing of each penetration subject to Type B testing. If opened following a Type A or Type B test, by leak rate testing the seal with gas at \geq Pa, 42 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Surveillance Requirements 4.7.A.2.b for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.6La.

3.0 LIMITING CONDITIONS FOR OPERATION

b. When Primary Containment Integrity is required, leakage rates shall be limited to:

1. An overall integrated leakage rate of less than or equal to LA, 1.2 percent by weight of the containment air per 24 hours at Pa, 42 psig.
2. A combined leakage rate of less than or equal to 0.6La for all penetrations and valves, subject to Type B and C tests when pressurized to Pa, 42 psig.
3. Less than one or equal to 46 scf per hour combined maximum flow path leakage for all main steam isolation valves when tested at 25 psig.

With the measured overall integrated primary containment leakage rate exceeding 0.75La, or the measured combined leakage rate for all penetrations and valves, subject to Type B and C testing exceeding 0.6La, or the measured combined maximum flow path leak rate exceeding 46 scf per hour for all main steam isolation valves, restore leakage rates to less than or equal to these values prior to increasing reactor coolant system temperature above 212°F or, alternatively, restore measure leakage rates to within these limits within one hour or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

b. The primary containment leakage rates shall be demonstrated in accordance with the schedule, criteria, methods, and provisions of 10 CFR Part 50, Appendix J as modified by approved exemptions:

1. Deleted
2. Deleted
3. Deleted
4. Deleted
5. Main steam isolation valve tests shall be conducted with gas at ≥ 25 psig. A combined maximum flow path leakage rate of ≤ 46 scf per hour shall be demonstrated for all main steam line isolation valves.

3.0 LIMITING CONDITIONS FOR OPERATION

c. When Primary Containment Integrity is required, the primary containment airlock shall be operable with:

1. Both doors closed except when the airlock is being used, then at least one airlock door shall be closed, and
2. An overall airlock leakage rate of less than or equal to 0.05La at Pa or 0.007La at 10 psig.

With the primary containment airlock inoperable, maintain at least one airlock door closed and restore the airlock to Operable status within 24 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

c. The primary containment airlock shall be demonstrated operable:

1. By performing overall airlock leakage test in accordance with the schedule and criterion of 10 CFR Part 50, Appendix J, as modified by approved exemptions.
2. Deleted
3. At six month intervals by verifying that only one door can be opened at a time. If the airlock has not been used since the last door interlock test, this test is not required.

d. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.