

NRC Form 200 (7-83)										U.S. NUCLEAR REGULATORY COMMISSION APPROVED OWS NO. 3150-0104 EXPIRES: 6/3/95									
LICENSEE EVENT REPORT (LER)																			
FACILITY NAME (1) <div style="font-size: large; font-family: monospace;">Monticello</div>										DOCKET NUMBER (2) <div style="font-size: large; font-family: monospace;">0 5 0 0 0 2 6 3</div>					PAGE (3) <div style="font-size: large; font-family: monospace;">1 OF 02</div>				
TITLE (4) <div style="font-size: large; font-family: monospace;">Crack Indications on Primary System Pressure Boundary Piping</div>																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME			DOCKET NUMBER(S)							
02	08	84	84	011	00	03	19	84				0 5 0 0 0							
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																	
N		20.405(a)				20.405(e)				90.73(a)(2)(iv)				72.71(b)					
POWER LEVEL (10)		20.405(a)(1)(i)				90.36(a)(1)				90.73(a)(2)(v)				72.71(e)					
01010		90.405(a)(1)(R)				90.36(a)(2)				90.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 356A)					
		20.405(a)(1)(ii)				90.73(a)(2)(i)				90.73(a)(2)(vii)(A)									
		20.405(a)(1)(iv)				90.73(a)(2)(ii)				90.73(a)(2)(viii)(B)									
		20.405(a)(1)(v)				90.73(a)(2)(iii)				90.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12)																			
NAME										TELEPHONE NUMBER									
Michael Keller, Production Engineer										612 295-5151									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC										
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)					NO														
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																			
<p>Crack indications were found in two systems:</p> <ol style="list-style-type: none"> 1. Recirculation System Sensing Line Penetration Piping 2. Residual Heat Removal <p>The degraded piping will be replaced with IGSCC resistant material.</p>																			
IE22																			

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Monticello	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	0 1 1	0 0	0 2	OF 0 2

TEXT (if more space is required, use additional NRC Form 306A's) (17)

On February 18, 1984 and February 23, 1984 crack indications were detected on the B Loop Jet Pump (P) Instrument Seal (SEAL) and on both A&B Loops of the Residual Heat Removal (RHR) Low Pressure Coolant Injection (LPCI) lines. At the time the cracks were detected the Reactor Mode Switch (HS) was in Shutdown. The crack indications were detected using Ultrasonic Examination methods.

B Loop Jet Pump (P) Instrument Seal (SEAL) crack indications are located within the heat affected zone of a weld between two A336 Stainless Steel eccentric reducers. Indications appear to be on the inside surface oriented parallel to the pipe axis.

Crack indications on the A&B Loops of the RHR LPCI lines are located on the reactor side of the stainless steel LPCI testable check valves (V), AO 10-46 A/B in the heat affected zone of welds joining a stainless steel pup piece to the valve and carbon steel piping.

For both the B Loop Jet Pump (P) Instrument Seal (SEAL) and the RHR LPCI, no through-wall cracks were detected. Crack indications are attributed to Inter Granular Stress Corrosion Cracking (IGSCC). The affected materials will be replaced with IGSCC resistant material.

There have been two previous similar occurrences:

M-RO-82-13 Recirc System IGSCC Problems

AO-75-03 Recirc Pump Bypass Line IGSCC Problems

Because all piping was intact and no through-wall cracks were detected, there were no radioactive materials released and no personnel exposures or injuries took place. There was no effect upon public health or safety.



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March 19, 1984

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Crack Indications on Primary System Pressure Boundary Piping

The License Event Report for this occurrence is attached.

These events were reported via the Emergency Notification System per 10 CFR Part 72 on February 18, 1984 and February 23, 1984.

David Musolf
Manager - Nuclear Support Services

DMM/js

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

IE22
1/1