

**PECO ENERGY COMPANY
PEACH BOTTOM ATOMIC POWER STATION
UNIT 2 AND 3
DOCKET NOS. 50-277 AND 50-278**

ANNUAL 10 CFR 50.59 REPORT

JANUARY 1, 1994 THROUGH DECEMBER 31, 1994

TABLE OF CONTENTS

CCN 95-14012

9505050029 950426
PDR ADDCK 05000277
R PDR

PECO ENERGY COMPANY
PEACH BOTTOM ATOMIC POWER STATION
UNIT 2 AND 3
DOCKET NOS. 50-277 AND 50-278
ANNUAL 10 CFR 50.59 REPORT

TABLE OF CONTENTS

<u>Miscellaneous Safety Evaluations</u>	<u>Page</u>
Engineering Change Request P946913	1
Engineering Change Request P948319	1
Engineering Change Request P948320	1
Engineering Change Request P949058	1
Engineering Work Request A0793597	1
Engineering Work Request A0867997	2
16-VRR-1 Relief Request	2
2D Residual Heat Removal Hatch Breach	2
ABB Qualifications	2
ARTS MELLA	3
Containment Atmospheric Dilution N2 Use During Operations	3
Control Rod To Exceed 56% Depletion	3
Core Design Report	3
Core Operating Limits Report	4
Core U/3 CYCLE 10 Core Operating Limits Report	4
Core U/3 CYCLE 10 Core Operating Limits Report	4
Elimination of Licensee Event Report Commitment	4
Feedwater Temperature 55 degree F Reduction	5
Feedwater Temperature 20 degree F Reduction	5
Fuel Bundle Movement	5
Logic System Functional Testing Performance at Power	5
Low Level Radwaste Facility	6
Nuclear Engineering Department Organization	6
Offgas Troubleshooting leak	6
Operation of P1	6
Operation of P1 in Symmetric Mode	7
POWER RERATE	7
Performance of P1s	7
Transverse-Incore-Probe # 2 Out of Service	7
Unit 1 Diesel Generator Removal from UFSAR	8
Unit 1 Security Fence Changes	8
Coast Down Operating Map	8
Dome Pressure during Coastdown	8
Control Rod Boron Depletion	9
Operation of Low Level R/W Facility.	9
 <u>Modifications</u>	 <u>Page</u>
MOD 0887	9
MOD 1106B	9

Modifications (continued) Page

MOD 1843	10
MOD 5130	10
MOD 5151	10
MOD 5169	10
MOD 5173	10
MOD 5177	11
MOD 5194	11
MOD 5195	11
MOD 5207	12
MOD 5232	12
MOD 5236	12
MOD 5248	12
MOD 5252	13
MOD 5269	13
MOD 5274	13
MOD 5280	13
MOD 5281	14
MOD 5290	14
MOD 5344	14
MOD 5347	14
MOD 5354	15
MOD 5357	15
MOD 5360	15
MOD 5370	15
MOD 5371	16
MOD 5374	16
MOD 5375	16
MOD 5383	16
MOD 5387	17
MOD 5396	17
MOD 5397	17
MOD P000128	17
MOD P000190	18
MOD P000207	18
MOD P000243	18
MOD P000270 (5362)	18
MOD P000287	19

Non Conformance Reports Page

NCR P000559	19
NCR P000561	19
NCR P920317	19
NCR P930326	19
NCR P930741	20
NCR P930809	20
NCR P930842	20
NCR P930882	20

Non Conformance Reports (Continued) Page

NCR P940055	21
NCR P940079	21
NCR P940150	21
NCR P940218	21
NCR P940252	22
NCR P940278	22
NCR P940344	22
NCR P940374	23
NCR P940868	23

Station Events Page

Performance Enhancement Program I0001332	23
Event Investigation Report 2-93-279	23

Procedures Page

PROCEDURE A-12.1	24
PROCEDURE AO-058C.1-2	24
PROCEDURE AO-063L.1	24
PROCEDURE ERP-101	24
PROCEDURE Emergency Plan	25
PROCEDURE Emergency Plan	25
PROCEDURE Emergency Plan	25
PROCEDURE M-57-013	25
PROCEDURE MAT-2254B	26
PROCEDURE ON-125 & BASES	26
PROCEDURE RT-W-20A-960-2(3)	26
PROCEDURE SP-1368	26
PROCEDURE SP-1458	27
PROCEDURE SP-2014	27
PROCEDURE ST-C-095-835-2	27
PROCEDURE ST-O-052-110-2 TC 94-884	27
PROCEDURE ST-O-052-110-2 TC 94-885	27

Temporary Plant Alterations Page

TPA 2-12-008	28
TPA 2-12-008	28
TPA 2-18-002	28
TPA 2-23-011	28
TPA 2-23-012	29
TPA 2-23-013	29
TPA 2-33-005	29
TPA 2-33-008	29
TPA 2-37-037	29
TPA 2-37-039	30

<u>Temporary Plant Alterations (Continued)</u>	<u>Page</u>
TPA 2-50-033	30
TPA 2-52-016	30
TPA 2-60-019	30
TPA 2-63-011	31
TPA 3-01-041	31
TPA 3-02-023	31
TPA 3-02-025	31
TPA 3-06-017	32
TPA 3-08-019	32
TPA 3-16-001	32
TPA 3-23-010	32
TPA 3-23-011	33
TPA 3-62-045	33

**PECO ENERGY COMPANY
PEACH BOTTOM ATOMIC POWER STATION
UNIT 2 AND 3
DOCKET NOS. 50-277; 50-278**

**ANNUAL 10 CFR 50.59 REPORT
JANUARY 1, 1994 THROUGH DECEMBER 31, 1994
SAFETY EVALUATION SUMMARIES**

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

ECR P946913

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This change permanently removed the spool piece on the Unit 2 head spray piping. The piping was replaced with two blind flanges. This change affected a drawing in the Updated Final Safety Analysis Report. This activity did not create any new adverse accidents or safety concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

ECR P948319 & P948320

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

These items changed the setpoint of the low differential pressure switch setpoints for the backup instrument nitrogen to Automatic Depressurization System from 0.0 psid to 20.0 psid. This change affected documentation specified in the Updated Final Safety Analysis Report. No new accidents or operating modes or adverse safety conditions were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

ECR P949058

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This activity reviewed and evaluated the use of a new type Reactor Pressure Vessel head strong back / carousel and accessories for optional use during a recent refueling outage. The use of this unit was not consistent with the system description specified in the Updated Final Safety Analysis Report. This change has been fully analyzed and did not create any new or different type accident or malfunction. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

EWR A0793597

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation supports the temporary breaching of one of the Main Control Room doors. This was done to support maintenance activities associated with the Main Control Room chillers. The change affected door closure requirements as described in the Updated Final Safety Analysis Report. This condition was evaluated to ensure that the Main Control Room would still be habitable under design events. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

EWR A0867997

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justified Emergency Core Cooling Systems, Reactor Core Isolation Cooling, and Diesel Generator operability for river temperatures up to 93 degrees F. The Updated Final Safety Analysis Report specifies that the design inlet temperatures for the Residual Heat Removal heat exchanger be no more than 90 degrees F. The activity will not create any new accidents or adversely affect the consequences of any existing or new type accident. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

16-VRR-1

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This safety evaluation revised the In Service Testing (IST) Program Specification (M-710) to add a forward exercise test for the Automatic Depressurization System Instrument Nitrogen check valves. This new testing provided additional insurance that these check valves would function properly in the forward direction. This change affected the IST Program which is addressed in the Safety Analysis Report. The testing is performed while the system is not required to be operable so no new or additional adverse conditions were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

2D Residual Heat Removal Hatch Breach

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Safety Evaluation reviewed the temporary breaching of the Unit 2 Secondary Containment to allow the replacement of the "2D" Residual Heat Removal Pump Motor. This change affected the Secondary Containment system descriptions as specified in the Updated Final Safety Analysis Report. The condition was evaluated and considered to be of minimal consequence based on the additional compensatory actions taken. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

ABB Qualifications

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justifies operation of Unit 2 during cycle 10 for a limited time with ABB Qualification Fuel Bundles leading the core in the MAPLHGR thermal limit during steady state operation. This activity affected correspondence between PECO and the NRC. The change did not adversely affect plant operations or safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

ARTS MELLA

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This evaluation justified the change to the maximum extended load line limit analysis and the "Average Power Range Monitor, Rod Block Monitor, Technical Specification" (ARTS) Improvement Program Analysis for Unit 3 cycle 10. The change affected documentation addressed in the Safety Analysis Report. No new adverse safety concerns were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

CAD N2 Use During Operations

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justified the use of the Containment Atmospheric Dilution system for Nitrogen makeup during normal operations. This activity affected the system description as specified in the Updated Final Safety Analysis Report. The change did not adversely affect normal operations or the system's functions during an accident. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Control Rod To Exceed 56% Depletion

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation justified the operation of Unit 2 in cycle 10 with control rod 30-23 (A163) at an exposure up to 56% B-10 depletion on the second segment. This change affected depletion limits in NEDE-30931-2-P. Operation in this manner did not result in exceeding the control rod's end of life B-10 equivalent depletion limit. Therefore, no new accidents or consequences were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Core Design Report

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation addressed the Unit 2 Core Design Report for Cycle 11 operations. The core load was of standard reload fuel and designed to be compatible with the existing fuel in the reactor. There was no impact on safety or increase in the probability of failure. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

Core Operating Limits Report

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation was revised to address the Unit 2 CORE OPERATING LIMIT Report for CYCLE 10 operations. The revision was made to meet the commitment to the NRC that the Quality Fuel Bundles will not lead the core with respect to thermal limits. This activity affected documentation addressed in the Safety Analysis Report. The change did not adversely affect any safety limits or operating modes. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Core U/3 CYCLE 10 COLR

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This evaluation justified the changes to the Unit 3 cycle 10 Core Operating Limit Report regarding MAPLHGR limits for all GE-8B fuel. The activity affected documentation addressed in the Safety Analysis Report. No new adverse reactivity or plant safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Core U/3 CYCLE 10 COLR

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This evaluation made a revision to the Unit 3 cycle 10 Core Operating Limit Report to relax Rod Block Monitor setpoints. The activity affected analysis documented in the Safety Analysis Report. No new reactivity or safety concerns were initiated as a result of the change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Elimination of LER Commitment

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This activity eliminated a commitment which was created as a result of on LER in 1991. The commitment was to perform High Pressure Coolant Injection and Reactor Core Isolation Cooling Exhaust line draining after turbine operations. Based on operating data and component performance, the draining of the exhaust line is not required and can be eliminated from the surveillance tests. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

F/W Temperature 55 degree F Reduction

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This safety evaluation allowed operation of Unit 2 with reduced feedwater temperatures of up to 55 degrees F prior to and during cycle extension. This change affected documentation specified in the Safety Analysis Report. The activity did not create any adverse operating modes or conditions or adversely affect plant safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

F/W Temperature 20 degree F Reduction

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justified the operation of Unit 2 during cycle 10 with a reduced feedwater temperature of up to 20 degrees F prior to cycle extension operation. This change affected Updated Final Safety Analysis Report section 14 transients. No new adverse concerns or operating modes were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Fuel Bundle Movement

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation addressed the movement of a fuel bundle with only 7 of the 8 tie rods intact. This activity affected system descriptions as described in the Updated Final Safety Analysis Report. No new accidents, transients, or adverse safety concerns were created during this evolution. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

LSFT Performance at Power

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This safety evaluation justified that the performance of Logic System Functional Testing at power is acceptable. This activity affected a commitment that specified that testing should be done while in cold shutdown. No new adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPC:RT

Low Level R/W Facility

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This review evaluated the handling and movement of waste from the area where it is received in processed and packaged from, to placement into storage at the facility. These activities affect the facility as described in the Updated Final Safety Analysis Report. Facility operations will not affect the operating units in any way. Based on the Safety Evaluation and the above information, it was determined that these activities did not constitute an Unreviewed Safety Question.

NED Organizational Changes

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation reviewed the Nuclear Engineering Department organizational changes at Chesterbrook. The changes involved the shifting of responsibilities to other organizations. This activity impacted wording in the Updated Final Safety Analysis Report chapter 13. These changes do not make any physical changes to the station and did not impact plant safety. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

Offgas Troubleshooting Leak

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

The safety evaluation justified Off Gas system manipulations in order to identify the source of a leak. This activity operated the Off Gas system in a manner not specified in the Updated Final Safety Analysis Report. No new accidents, transients, or adverse operating modes were created during the performance of this troubleshooting. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Operation of P1

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation justified the operation of Unit 2 during cycle 10 with the P1 program in symmetric mode with Group 18, 20, and 21 asymmetric by no more than eight positions (four notches). This activity affected system operations as described in the Updated Final Safety Analysis Report section 3.6. Plant response to a transient remained bounded by the safety analyses. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

Operation of PI in Symmetric Mode

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation justified the operation of Unit 2 during cycle 10 with the P1 program in symmetric mode with groups 18, 20, and 21 asymmetric by no more than six positions. This allowed asymmetric control rod pattern operation with the leakage suppression control rods inserted. This change affected section 3.6.5 of the Updated Final Safety Analysis Report. No new adverse concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Power Rerate

Year Implemented:

Unit 2 (1994)

Unit 3 (not implemented yet)

This evaluation justified the revision of a General Electric Nuclear Energy document Power Rerate Licensing Report with respect to "Average Power Range Monitor, Rod Block Monitor, Technical Specification" (ARTS). The change affected documentation specified in the Safety Analysis Report. The activity did not adversely affect plant safety or any operating modes or accidents. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Performance of P1s

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation justified the operation of Unit 2 during cycle 10 with the P1 Program in symmetric mode with groups 18, 20, 21, and 26 asymmetric by no more than 8 positions (4 notches). The activity affected the systems operational descriptions as specified in the Updated Final Safety Analysis Report. No adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TIP # 2 Out of Service

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This evaluation justifies the operation of Unit 3 during Cycle 10 with the "B" Traversing In core Probe (TIP) out of service. The indexer for the "B" TIP was not capable of performing its function to allow detector access to portions of the core to determine local power distributions. Symmetric TIP location traces are used as substitute values. The use of asymmetric control rods are to be reviewed on a case by case basis. No new adverse safety concerns were created as a result of the activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

U/1 D/G Removal from UFSAR

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation reviewed the removal of all references to the Unit 1 Diesel Generator in the Updated Final Safety Analysis Report. The diesel was used as a backup electrical power supply to the Unit 1 Technical Support Center (TSC). This was done due to the recent completion of a modification (5396) which installed a new backup electrical power source to the Unit 1 TSC. The new power supply is for Station Blackout Requirements and is fed from a nearby hydroelectric plant. No new adverse safety or electrical concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

U/1 Security Fence Change

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This 50.59 review addresses the proposed change to move the Unit 1 containment security fence to allow for training activities in the southern lay down area adjacent to the containment structure in the Unit 1 facility. This change allows for the security fence to be removed with no degradation in security at Unit 1. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

U/2 Coast Down Operating Map

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation justified expanding the allowable operating domain of the power / flow map on Unit 2 during cycle 10 coast down. This activity affected the allowable ranges for operation as described in the Updated Final Safety Analysis Report. This change did not reduce the margin of safety or adversely affect any accident initiators and no adverse safety conditions were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

U/2 Dome Pressure during CD

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This safety evaluation justified constant reactor steam dome pressure of 1055 psig during Unit 2 cycle 10 end of cycle coastdown operations. This was done to maximize total energy output at this point in the cycle. Operations in this mode is different than as specified in the Updated Final Safety Analysis Report. No adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

Control Rod Boron Depletion

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation allowed control rod blade A163 located at position 30-23 to exceed 56% boron-10 depletion on Unit 2 for cycle 10. This activity affected boron depletion limits as specified in the Safety Analysis Report. Fuels evaluated this condition and determined that it did not adversely affect plant safety or reduce the margin of safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Operation of Low Level R/W Facility

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation reviewed the current operating practices of the Low Level Radwaste Storage Facility at Peach Bottom. The review involved the handling and movement of waste in the facility. The process affected UFSAR Section 9.3.4. No new safety concerns or possible unplanned exposure or releases were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that these operating practices did not constitute an Unreviewed Safety Question.

MOD 0887

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification upgrades the Reactor Recirculation A & B speed control loops, scoop tube positioner, master controller, dual speed controller, speed indication circuits, and the reset logics. The enhancements improved operation and reliability. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 1106B

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification reworked the condensate phase separator interlaces and internals to minimize plugging and improve resin processing operations. This activity affected documentation addressed in the Safety Analysis Report. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 1843

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification replaced the existing Reactor Feed Water Control system with an digital control type system and performed post modification testing. This activity affected the systems operation as described in the Updated Final Safety Analysis Report. No new adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5130

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification upgraded the existing Containment Atmosphere Dilution instrumentation loops. It also replaced instruments in the Containment Atmosphere Control instrumentation loops. The upgraded components more accurately measure the loop processes. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

MOD 5151

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification provided a time delay in the high flow isolation signal to the solenoid valves (SV) in the nitrogen supply lines to the Automatic Depressurization System Safety Relief Valves. The piping downstream of the SVs depressurizes with time due to piping leakage. When the SVs are opened, the initial rush of nitrogen to repressurize the piping automatically closes the SVs due to excessive high flow. The time delay will provide enough time to repressurize the piping without an unwanted isolation of the SVs and ensure the a large piping leak would be adequately isolated. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5169

Year Implemented:

Unit 2 (1993)

Unit 3 (1994)

This modification replaced the eight existing Exide battery chargers with new seismically qualified Class 1E charger assemblies. The existing chargers were approaching the end of their life. This activity enhanced the system's reliability. The change affected figures addressed in the Updated Final Safety Analysis Report and no adverse safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5173

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification converted the Standby Gas Treatment system charcoal filter deluge system from automatic operation to manual operation. The new system consists of a manual block valve in an accessible location. The object of the modification was to minimize the risk of inadvertent operation of the deluge system. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5177

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification relocated the Automatic Depressurization System annunciator windows in the Main Control Room to improve human factors. This change affected documentation addressed in the Updated Final Safety Analysis Report. The modification did not adversely affect plant operations or create any safety concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5194

Year Implemented:

Unit 2 (1994)

Unit 3 (not implemented yet)

This modification installed a pressure switch in the Shutdown Cooling Valve (MU-17) to prevent an unwanted spurious opening during power operation in the event of an Appendix R fire. The change did not create any new adverse safety concerns or operating modes. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5195

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification provided a permanent instrument loop that is capable of an enhanced display of Reactor Water level during Refueling operations. This change affected documentation specified in the SAR. No new refueling safety issues or concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5207

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification upgraded the Raw Water and Domestic Water Systems. This change affected the system descriptions as specified in the Updated Final Safety Analysis Report. No new adverse plant operating conditions or safety concerns were introduced by this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5232

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification installed new storage racks in the Low Level Radwaste Building. This activity will provide increased storage capabilities and the use of these racks does not create any adverse conditions that would effect plant operations or radiation exposures. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5236

Year Implemented:

Unit 2 (1992)

Unit 3 (1994)

This modification was installed to provide a hardened torus vent in accordance with Generic Letter 89-16. The main objective of the vent is to mitigate the consequences of a long term loss of decay heat removal. This was beyond the plant licensing basis and assumes, with the exception of the Residual Heat Removal system, that all other systems are operational and the core is not in a degraded condition. The design of the torus hardened vent has been analyzed in accordance with 10CFR50.59 and Generic Letter criteria. No safety concerns were created. This change was made to comply with an NRC commitment and provides plant safety improvements in an outside of design bases event. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5248

Year Implemented:

Unit 2 (not implemented yet)

Unit 3 (1994)

This modification removed the area radiation monitor from service from the new fuel storage vault. The fuel storage vault has never been used since new fuel is placed directly into the fuel pool. This change affected documentation addressed in the Updated Final Safety Analysis Report. No new accident or adverse safety concerns were created during the modification. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5252

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification upgraded the radiation monitoring capability of the HPSW radiation monitoring system. This activity provided a more reliable system. The change affected figures addressed in the Updated Final Safety Analysis Report and no adverse safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5269

Year Implemented:

Unit 2 (1993)

Unit 3 (1994)

This modification added a new non safety related DC electrical system to replace the existing BOP battery. It also moved non safety related loads from the safety related DC electrical systems to the new non safety related DC systems. This change affected documentation specified in the Updated Final Safety Analysis Report. No safety concerns were introduced as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5274

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification replaced the existing CAC/CAD analyzers with improved instrumentation to improve operations and reliability. The activity made the system operation and layout different from that described in the Updated Final Safety Analysis Report. No safety concerns were introduced as a result of this activity. Based on the Safety Evaluation and the above information it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5280

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification replaced the existing seismic monitoring equipment with an improved system. The new system is functionally equivalent but provides improved operation and reliability. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5281

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification replaced the existing Main Control Room Ventilation radiation monitoring system with an upgraded and more reliable system. The existing system was obsolete. This activity affected documentation addressed in the Safety Analysis Report and no adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5290

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification installed safety related indicators located within several Reactor vessel pressure instrumentation loops to verify loop functionality. Station technicians in the past needed to perform lengthy tests to determine loop functionality to satisfy the Technical Specification requirements. This change eliminates these tests since the operators can routinely survey the indicators during their normal shift checks. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5344

Year Implemented:

Unit 2 (1992)

Unit 3 (1994)

This modification added vibration instrumentation to the High Pressure Coolant Injection turbine-pump assembly. This instrumentation does not affect the pressure retaining boundaries of the High Pressure Coolant Injection (HPCI) system or operational modes. The data available from this instrumentation was incorporated into the HPCI Surveillance Testing so that the origin of HPCI pump vibration can be determined. This activity will enhance system testing. There is no impact on system capability or operation. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5347

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

The modification installed improved flow meters at several locations on the Emergency Service Water system. These flow meters are used to provide a means of determining the flow rates at various locations to support testing and maintenance activities. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5354

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification was designed to provide improved chlorine delivery in the Water Treatment System and added improved chlorine monitoring. This activity affected documentation addressed in the Updated Final Safety Analysis Report. No new adverse safety concerns or new adverse operating conditions were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5357

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification provided remote operation to enable transfer of liquid radwaste from the water collector tank by the addition of a Air Operated Valve instead of a Manually Operated Valve. In addition, a manual valve was installed on the Floor Drain Collector Tank to facilitate easier liquid transfers. This change affected system operations as specified in the Updated Final Safety Analysis Report. The activity did not create any new adverse operating modes or accidents. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5360

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification added a new Service Water Bay chemical injection system to treat the intake structure for macro fouling organisms before excessive growth occurs. The new equipment was permanently installed to replace the portable system which was previously in use. This activity affected figures addressed in the Updated Final Safety Analysis Report and no adverse safety concerns were introduced. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5370

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Reactor Water Cleanup system modification replaced the three existing 50% capacity pumps with two 100% capacity pumps and upgraded the existing filter demineralizer precoat cycle control circuitry and manual isolation valves. It also installed new piping to support the change. This modification improved the Reactor Water Cleanup System by providing better overall system performance. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5371

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification provided the Main Generator with water in leakage detection equipment. The new instrumentation installed monitors the dew point of the hydrogen gas in the generator. This activity affected documentation addressed in the Updated Final Safety Analysis Report. The change is an enhancement during generator operations and did not adversely affect plant safety or operations. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5374

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification allows the operation of the plant to include the extended operating region bounded by the rod line which passes through the 100% power / 75% core flow point (approximately 121% rod line). The technical analysis is referred to as the Maximum Extended Load Line Limit (MELLLA) analysis. In addition, this modification also incorporated the Average Power Range Monitor, Rod Block Monitor, and Technical Specification Improvement (ARTS) Program which increases plant operating efficiency by updating the thermal limits requirements and improving plant instrumentation responses and accuracy. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5375

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification enhanced several alarm annunciator logics to allow no alarms to be up during normal operations. Five alarm annunciators were normally up during power operations. This change allowed the alarms to be cleared during power operations. This affected a logic figure in the Updated Final Safety Analysis Report. This change does not create any new adverse operation conditions or adverse safety concerns. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

MOD 5383

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification added a manual block valve in the equalizer line of the Residual Heat Removal System testable check valve, which is a containment isolation valve. This will allow for the performance of a Local Leak Rate Test to positively determine whether the testable check valve or equalizer valve is leaking. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD 5387

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification replaced the existing Drywell Leak Detection System with an improved and more reliable type monitor. The change affected documentation addressed in the Updated Final Safety Analysis Report. No new adverse safety concerns or new operating modes / transients were introduced by this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5396

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification installed an addition bus and associated switchgear to support station black out. This activity affected documentation specified in the Safety Analysis Report. No new accidents, transients, or adverse operating modes were created as a result of this change and electrical reliability was increased as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD 5397

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification provided various design changes to the Emergency Diesel Generators. These enhancements involved the lube oil, auxiliary inter cooler, jacket coolant pump, and the air start systems. No safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

MOD P000128

Year Implemented:

Unit 2 (1994)

Unit 3 (1993)

This modification was installed to provide a continuous backfill system to the reference legs associated with Reactor Water Level Instrumentation. This activity will enhance level indication reliability. This change affected documentation addressed in the Safety Analysis Report. No adverse safety concerns or new operating modes were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD P000190

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This modification replaced seven valve motor actuators with larger type units. This was done to meet increased thrust demand requirements. The change affected documentation addressed in the Updated Final Safety Analysis Report. No new adverse safety concerns were created during this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD P000207

Year Implemented:

Unit 2 (1994)

Unit 3 (not implemented yet)

This modification changed the Feedwater / Recirculation Flow Control system instrumentation, alarms, and runback logics. The change affected documentation specified in the Updated Final Safety Analysis Report section 7.9. The activity did not adversely affect plant operations or accident analysis. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD P000243

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This modification installed an equalization line and access platform for the Diesel Driven Fire Pump test header valve. This activity affected documentation addressed in the Updated Final Safety Analysis Report. No new adverse safety concerns or operating modes were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

MOD P000270 (5362)

Year Implemented:

Unit 2 (1994)

Unit 3 (not implemented yet)

This modification upgraded the Reactor Pressure Vessel Bottom Head Drain line to improve the accuracy of the line temperature measurement to minimize unnecessary shutdowns, power reductions and stratifications of the bottom head. This activity affected documentation addressed in the Updated Final Safety Analysis Report. This change is an enhancement and does not create any new adverse safety concerns or new plant transients or accidents. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

MOD P000287

Year Implemented:

Unit 2 (1994)

Unit 3 (not implemented yet)

This modification revised reactor water level measurement pressure compensation to account for increased operating pressures and increased ambient temperatures in the drywell and reactor building. This change affected documentation specified within the Safety Analysis Report. No new adverse system operations or new concerns were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P000559 & P000561

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

These Non Conformance Reports identified and resolved discrepancies between the plant and the Fire Protection Program figures. This activity made the figures match the installed configuration. No new adverse safety concerns were created and no new operating modes were created. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

NCR P920317

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report evaluation addressed what maintenance activities are required on the spare Emergency Core Cooling System Room Coolers to ensure Environmental Qualifications are maintained. This change affected documentation addressed in the Updated Final Safety Analysis Report. The activity did not affect plant operations or safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P930326

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report evaluated the action levels associated with a river flooding scenario. The action levels were lowered by 2 feet due to the design of the Circulating Water Pump Structure. The structure was not designed to support the current action levels. This change affected documentation in the Updated Final Safety Analysis Report. This change is conservative and did not affect plant safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

NCR P930741

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report changed the Motor Driven Fire Pump Logic to prevent it from automatically loading on to the EDG on a loss of power condition. This change affected documentation addressed in the Fire Protection Plan. This activity did not create any new adverse safety concerns or adverse plant safety conditions. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P930809

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation was performed to support the weld repair of the Emergency Service Water Booster Pump piping. This activity affected system descriptions as specified in the Updated Final Safety Analysis Report. System line up and operations have been evaluated and no adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P930842

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report addressed a change to the Emergency Service Water flow criteria for the Emergency Core Cooling System and Reactor Core Isolation Cooling Room Coolers. The change affected documentation previously specified in the Updated Final Safety Analysis Report. The change did not adversely affect plant operations, safety, or accident analysis. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P930882

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report addressed discrepancies between the as found condition and the "E-1305 Panel Electrical Schedules". The Updated Final Safety Analysis Report was affected by this condition. An evaluation was performed which ensured that the as found conditions were acceptable and that electrical margins were still acceptable. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION
UNIT 2 & 3
Docket No. 50-277 & 50-278
1994 10 CFR 50.59 REPORT

NCR P940055

Year Implemented:
Unit 2 (1994)
Unit 3 (1994)

This Non Conformance Report allowed the use of Bussman JHC100 fuses in a Motor Control Center which supplies power to a electrical distribution panel (20B38-82). The fuses were installed as part of the initial Appendix R modification. This change affected fuse configurations as specified in the Fire Protection Plan. No new adverse safety concerns, accident modes, or equipment failures were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940079

Year Implemented:
Unit 2 (1994)
Unit 3 (N/A)

This Non Conformance Report removed the Unit 2 Torus Vacuum Breaker three degree position switches until they are permanently replaced with a new type in Refuel Outage 2R11. These switches are specifically addressed in the Updated Final Safety Analysis Report. Removal of these switches will not adversely affect plant safety or create any new accident or transient initiators. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940150

Year Implemented:
Unit 2 (1994)
Unit 3 (N/A)

This Non Conformance Report replaced the 480 volt magnetic only type circuit breaker associated with a Radwaste area radiation monitor (20B060-21) to a thermal magnetic type breaker due to multiple high impedance fault considerations. This change affected protective device configurations as specified in the Fire Protection Plan. No new adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940216

Year Implemented:
Unit 2 (N/A)
Unit 3 (1994)

This Non Conformance Report evaluated a calculation associated with fuse configurations and required the installation of Bussmann FRS-R-150 fuses in series with several existing magnetic trip devices. This change affected fuse coordination figures located in the Fire Protection Program. No new accidents, transients, or adverse operating modes were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

NCR P940252

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report evaluated the Torus Narrow Range Level Transmitters design range of 13.7' to 15.7'. A recent survey indicated that level could only be measured on this instrument from 13.6' to 15.6', therefore, it could not be measured between 15.6' and 15.7'. The Non Conformance Report was dispositioned to use as is. This activity affected Torus water level ranges as described in the Safety Analysis Report. Since levels are normally maintained between 14.6' and 14.9', no new accidents, transients, or malfunctions are created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940278

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report evaluated changing the Diesel Generator voltage regulator setting range between 4160 and 4400 volts. This change affected system descriptions as specified in the Updated Final Safety Analysis Report. This activity did not adversely affect system operations, plant safety, or the possibility of a transient, accident, or malfunction of equipment. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940344

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This review addressed the use-as-is disposition of a Non Conformance Report. The Non Conformance Report identified three improper assumptions for the Emergency Diesel Generator loading tables in the Updated Final Safety Analysis Report. The calculations associated with these tables have been revised. This activity affected diesel loading calculations and its associated tables in the Updated Final Safety Analysis Report. This change did not create any new adverse electrical loading concerns and did not affect plant safety or reliability. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

NCR P940374

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Non Conformance Report evaluated and justified continued operation of the Unit 2 reactor with weld crack indications identified on the shroud. This activity made changes to the Updated Final Safety Analysis Report since initial review and approval were not based on these crack indications. Calculations and analysis performed on the as found conditions verified that no new adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

NCR P940868

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This modification installed the VOTES Torque Cartridge on MO-3-10-025A to perform monthly diagnostic testing. The activity removed the handwheel. The handwheel is described in the Updated Final Safety Analysis Report for this valve. This change did not adversely affect the operation of the valve or associated system during normal or emergency system operations. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Performance Enhancement Program I0001332

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This safety evaluation was written to evaluate the consequences of a missed fire watch. This condition resulted in a Technical Specification violation. The condition was temporary in nature due to a personnel error. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

Event Investigation Report 2-93-279

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation addressed an event where a high radiation door was not locked and secured as specified in the Technical Specifications. No plant safety or operating concerns were created as a result of this event. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

PROCEDURE A-12.1

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure revised the guidance involving actions for fire system impairments. The change affected the associated actions specified in the Fire Protection Program. This activity did not create any adverse safety condition or create any new or different concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE AO-058C.1-2

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This procedure provided the instruction needed for supplying temporary electrical power to lighting panel 95L from a motor control center. This was done to avert the loss of electrical power to Off Gas Stack Sampling systems and fire code devices during bus outage activities which de-energize the power source. This change affected electrical lineups as specified in the Updated Final Safety Analysis Report. No unsafe configuration was created as part of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE AO-063L.1

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure provided the steps needed for supplying temporary electrical power to the Division I Main Control Room radiation instrumentation and Drywell Radiation Instrumentation. The electrical system lineup was different than that specified in the Updated Final Safety Analysis Report. No new accident, transients, or operating modes were created by this lineup. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE ERP-101

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure was revised to provide references and clarifications to an Emergency Response Implementing Procedure. The activity affected documentation in the Updated Final Safety Analysis Report Appendix O. This change did not adversely affect plant safety or the station's ability to respond during an emergency. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

PROCEDURE Emergency Plan

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure was revised to address changes in the plant evacuation process and the removal of the Conference Center. Evacuating personnel will go to Unit 1 instead of the Conference Center. This change affected documentation in the Safety Analysis Report and NRC commitments. The activity did not adversely affect plant operations or Emergency evacuation processes. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE Emergency Plan

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justified revisions to address the Power Rerate Project. The change revised the plan to remove any references to the station's maximum power capacity which is described in the Updated Final Safety Analysis Report. No new adverse safety concerns were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE Emergency Plan

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation justified revisions to the Emergency Plan to clarify Emergency Response Organization personnel responsibilities and authorities, corrected typographical errors, and incongruities. This change affected the previous revision to the EP Plan. This activity did not adversely affect plant safety or personnel response during an emergency. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE M-57-013

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure provided the means to perform maintenance activities on safety related battery cells while maintaining the battery and DC power supply operable. This activity affected documentation specified in Updated Final Safety Analysis Report section 8. This procedure did not adversely affect plant operations during procedure performance. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

PROCEDURE MAT-2254B

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This evaluation supports the Modification Acceptance Testing (MAT) for installation of a third Off Site Electrical Power Source under Modification 2254. This activity is considered a Test not described in the Safety Analysis Report. This testing did not adversely affect plant operations and safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE ON-125 & BASES

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure provided the necessary instructions to restore Shut Down Cooling following an inadvertent Shut Down Cooling isolation while the reactor pressure head is removed. This activity affected the "Loss of Shut Down Cooling" scenario addressed in the Updated Final Safety Analysis Report. This change did not adversely affect any described accidents or transients. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE RT-W-20A-960-2(3)

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure change incorporated modification P000421 which unplugged floor drains in the plenums for the Turbine Building (TB), TB Equipment Cell, Reactor Building (RB), and the Refuel Floor (RF) supply vent fans. These drains were unplugged because of past water and ice accumulation which resulted in a significant slipping hazard for personnel. This change affected a 1981 commitment in response to IE Bulletin 80-10 and an Inspection Report 92-07. No new adverse plant safety conditions were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE SP-1368

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure allowed the operation of the Core Spray pumps with the Emergency Service Water flow to the motor oil coolers isolated. This was done to obtain test data to support elimination of the oil coolers. This change affected system descriptions regarding Core Spray motor oil coolers. Operation in this mode did not create any new concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

PROCEDURE SP-1458

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This procedure removed the 5th stage feedwater heaters from service on Unit 2 cycle 10 prior to cycle extension allowing a final feedwater temperature reduction of up to 55 degrees F. This change affected documentation specified in the Safety Analysis Report. Operation in this condition has been analyzed and no new adverse conditions were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE SP-2014

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This special test verified that the EHC control system can adequately control reactor pressure for power at a re-rated condition of 105%. This was a test not described in the Updated Final Safety Analysis Report. PECO and GE have completed a study and concluded that the Turbine Control System is adequate for operations at rerate conditions. Plant safety was not affected by the test. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE ST-C-095-835-2

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This evaluation revised a Surveillance Test to allow the use of portable composite sampling systems on the Circulating Water Intake and Discharge. These portable units were required due to equipment problems associated with the permanently installed systems. This change affected the system descriptions in the Updated Final Safety Analysis Report. The new systems are more reliable and do not create any adverse safety concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PROCEDURE ST-O-052-110-2 TC 94-884 & 885

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This procedure, Diesel Generator Simulated Automatic Actuation and Load Acceptance Test, was changed due to Unit 3 being shutdown with a Residual Heat Removal Shutdown cooling loop inservice. The test could not be adequately performed while in this configuration. A jumper was added to the circuit to facilitate test performance. This activity affected system descriptions as specified in the Updated Final Safety Analysis Report. This change did not create any new type transient or malfunction which would affect plant safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 2-12-008

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration removed a flow glass lens on the Reactor Water Clean Up system and installed a steel plate to stop a sealing surface from leaking. The steel plate will be in place until either a new flow glass or a straight pipe is installed. This change affected figures in the Updated Final Safety Analysis Report. This activity did not adversely affect system operations or create any new safety concerns. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-12-008

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration deleted the low flow, high flow, and high vibration trips associated with the "2A" and "2C" Reactor Water Clean Up pump trip logics due to spurious tripping. This configuration is different than that specified in the Updated Final Safety Analysis Report. The change will not create any new type accidents, transients, or malfunctions of equipment important to safety. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-18-002

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration was installed due to the use of the GE In-Core sipping hoods which necessitate the alteration of a refueling platform grapple interlock feature in order to ungrapple the hood when it is seated on the core. The interlock involved is one which senses the elevation of the main hoist when the platform is over the core. This change affected system configurations as described in the Updated Final Safety Analysis Report. This condition was analyzed and no new adverse safety concerns were created. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-23-

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration was installed to jumper several Motor Control Center start logic relays associated with the High Pressure Coolant Injection system. This activity affected documentation addressed in the Safety Analysis Report. This change does not create any new adverse safety conditions but will enhance High Pressure Coolant Injection component reliability. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 2-23-012

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration was installed to jumper several Motor Control Center start logic relays associated with the High Pressure Coolant Injection system. This activity affected documentation addressed in the Safety Analysis Report. This change does not create any new adverse safety conditions but will enhance High Pressure Coolant Injection component reliability. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-23-013

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration installed a temporary drain line on the High Pressure Coolant Injection system turbine drain pot due to a leaking valve. This change affected the system description of turbine draining as described in the Updated Final Safety Analysis Report. The change did not adversely affect system performance or accident mitigation. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-33-005 & 008

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

These Temporary Plant Alterations failed open solenoid valves which supplied cooling water to Emergency Core Cooling Systems, the Reactor Core Isolation Cooling system, and the Emergency Diesel Generators due to solenoid valve sticking problems. This was done until a permanent solenoid replacement is completed. This affected system operations as defined in the Safety Analysis Report. No new adverse safety concerns were created since all systems still function per their design. Based on the Safety Evaluation and the above information, it was determined that these changes did not constitute an Unreviewed Safety Question.

TPA 2-37-037

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration was installed to isolate a ground on the Unit 3 Main Transformer heat detection circuit from the plant DC power system. This change affected system description as specified in the Fire Protection Program. The activity does not adversely affect plant operations or the performance of any safety systems. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 2-37-039

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration was installed to isolate an electrical short on the Unit 3 Auxiliary Transformer heat detection circuit and prevent an inadvertent discharge. This change affected system performance as described in the Fire Protection Program. The activity did not adversely affect normal plant operations or the function of any safety system. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-50-033

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration was installed to allow the clearing of the ISOPHASE BUS TROUBLE alarm in the Unit 2 Control Room. This alarm was being brought in by a local HI HUMIDITY alarm at the bus. This activity temporarily defeated the HI HUMIDITY alarm to allow operators to be alerted to other potential isophase bus problems. This change affected the system description as specified in the Updated Final Safety Analysis Report. The Temporary Plant Alteration did not adversely impact any safety system actuations or plant operations. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-52-016

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration removed the E-1 EDG Scavenging Air Cooler Standby Valve disc since it had separated from the valves stem. The system will be restored to its normal configuration in a future outage. This activity affected documentation addressed in the Safety Analysis Report. The valve is maintained in its safety position and did not create any new adverse conditions. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 2-60-019

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration allowed the monitoring of various system parameters with temporary equipment during power ascension testing. This change affected system descriptions as specified in the Safety Analysis Report. Isolators have been installed as needed and no adverse safety concerns were created during this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 2-63-011

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration installed a strip chart recorder to address concerns associated with the Main Control Room Radiation Monitors experiencing spurious spiking. The equipment recorded system fluctuations from a steady state condition to be monitored. This activity affected documentation specified in the Safety Analysis Report. The change did not adversely affect system operations or plant safety in any way. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-01-041

Year Implemented:

Unit 2 (1994)

Unit 3 (N/A)

This Temporary Plant Alteration installed temporary push buttons in the Cable Spreading Room on the EHC pressure control circuits. This allowed the raising and lowering of the setpoint from this alternate location. This affected the system as described in the Updated Final Safety Analysis Report. No new adverse safety concerns were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-02-023

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration was installed on Unit 3 Recirculating Motor Generator Set to monitor various operational parameters. This activity affected documentation addressed in the Safety Analysis Report. No adverse safety concerns, new operating modes, or plant transients were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-02-025

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration was installed to defeat a false alarm associated with an Excess Flow Check Valve which is masking several other alarm circuits. This change affected system descriptions as specified in the Updated Final Safety Analysis Report. No new accidents, equipment failures, or safety concerns were created as a result of this activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 3-08-017

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration temporarily installed jumpers on Off Gas flow switches to prevent inadvertent Off Gas system isolations during an effort to repair a steam leak on the Off Gas / Recombiner system. This activity affected Off Gas system descriptions as specified in the Updated Final Safety Analysis Report. No safety concerns were introduced during this repair activity. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-08-019

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration provided an air increase into the Off Gas system to dilute off gas concentrations and potentially reduce the impact of off gas leaks on airborne activity. This change affected the system description as specified in the Updated Final Safety Analysis Report. No new accident modes or equipment failures were created as a result of this change. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-16-001

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration installed a test tee connection and a block valve on the instrument nitrogen compressor to support troubleshooting activities on the pressure rupture disc. This change affected figures specified in the Updated Final Safety Analysis Report. The Temporary Plant Alteration did not adversely affect plant safety or system operations. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-23-010

Year Implemented:

Unit 2 (1994)

Unit 3 (1994)

This Temporary Plant Alteration was installed to jumper several Motor Control Center start logic relays associated with the High Pressure Coolant Injection system. This activity affected documentation addressed in the Safety Analysis Report. This change does not create any new adverse safety conditions but will enhance High Pressure Coolant Injection component reliability. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

PEACH BOTTOM ATOMIC POWER STATION

UNIT 2 & 3

Docket No. 50-277 & 50-278

1994 10 CFR 50.59 REPORT

TPA 3-23-011

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration was installed to jumper several Motor Control Center start logic relays associated with the High Pressure Coolant Injection system. This activity affected documentation addressed in the Safety Analysis Report. This change does not create any new adverse safety conditions but does enhance High Pressure Coolant Injection component reliability. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.

TPA 3-62-045

Year Implemented:

Unit 2 (N/A)

Unit 3 (1994)

This Temporary Plant Alteration was installed to clear a faulty Control Rod Drift Alarm by the installation of a jumper. This change affected the system description as specified in the Updated Final Safety Analysis Report. The activity did not create any new adverse operating modes, accidents, or transients. Based on the Safety Evaluation and the above information, it was determined that this change did not constitute an Unreviewed Safety Question.