

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-0104
EXPIRES - 9/31/85

FACILITY NAME (1)										DOCKET NUMBER (2)										PAGE (3)																													
INDIAN POINT, UNIT 2										0 5 0 0 0 2 4 7										1 OF 0 3																													
TITLE (4)																																																	
INOPERABLE CONTAINMENT ISOLATION VALVES																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR				YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR				FACILITY NAMES										DOCKET NUMBER(S)										
0 1			0 4			8 4				8 4			0 0 1			0 0 0			2 0			3 8			4														0 5 0 0 0 1 1										
OPERATING MODE (9)										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																							
POWER LEVEL (10) 1,0,0										20.402(a)										20.406(a)										60.73(a)(2)(iv)										73.71(b)									
										20.406(a)(1)(ii)										60.36(a)(1)										60.73(a)(2)(v)										73.71(c)									
										20.406(a)(1)(iv)										60.36(a)(2)										60.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 364A)									
										20.406(a)(1)(vi)										60.73(a)(2)(ii)										60.73(a)(2)(viii)(A)																			
										20.406(a)(1)(vii)										60.73(a)(2)(iii)										60.73(a)(2)(vii)(B)																			
20.406(a)(1)(viii)										60.73(a)(2)(iv)										60.73(a)(2)(ix)(B)										60.73(a)(2)(x)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME										MICHAEL BLATT - DIRECTOR REGULATORY AFFAIRS										TELEPHONE NUMBER																													
																				AREA CODE																													
																				9 1 4										5 1 2 6 1 5 1 1 2 7 1																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																					
X		JIM F		ISV		A14		9/9		Y																																							
E		JIM		ISV		E1		12/7		Y																																							
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO										EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																			
ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)																																																	
<p>During normal power operation, a low pressure alarm on the Weld Channel and Containment Penetration Pressurization System for the V.C. Ventilation annunciated in the Central Control Room. The weld channel flow fluctuated and finally decreased below normal flow. SOV-1278 (the air supply to the Containment Penetration between the Containment Purge Exhaust Valves) had malfunctioned and closed. SOV-1278 was replaced but penetration pressurization still could not be maintained. Leakage past one or both Containment Purge Exhaust Valves apparently caused the pressure loss. A plant shutdown was commenced. A subsequent investigation revealed that there was leakage past FCV-1172 valve seat and that FCV 1173 appeared to be slightly open. The tee-ring seat and compression ring were replaced on FCV-1172 and FCV-1173 was cycled several times and left in the closed position. The valves were successfully tested and returned to service. The plant was restarted and returned to service on 1/7/84.</p>																																																	
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0 5 0 0 0 2 4 7 8 4 -	0 0 1 -	0 0	12	OF 3

TEXT (if more space is required, use additional NRC Form 366A (17))

On January 4, 1984 at 1:00 PM a low pressure alarm on the Weld Channel and Containment Penetration Pressurization System (WCCPPS) to the V.C. Ventilation annunciated in the Central Control Room.

The Containment Penetration and Weld Channel Pressurization System delivers a regulated supply of clean and dry compressed air from either of the plant's 100 psig compressed air systems located outside the containment to all containment penetrations and inner liner weld channels. The system maintains a pressure in excess of containment design pressure continuously during all reactor operations thereby ensuring that there will be no out-leakage of the containment atmosphere through the penetrations and liner welds during an accident. The primary source of air for this system is the instrument air system. Two instrument and control air compressors are used, although only one is required to maintain pressurization at the maximum allowable leakage rate of the pressurization system. Containment penetrations and liner weld channels are grouped into four independent zones to simplify the process of locating leaks during operation.

As a result of the alarm condition plant personnel investigated the different possibilities in order to identify the cause of the low pressure and erratic flow condition. These investigations revealed that SOV-1278 had malfunctioned and that there was leakage past one or both Containment Purge Exhaust Valves (FCV-1172 and FCV-1173).

Each ventilation purge duct penetration is provided with two tight-closing butterfly valves, which are normally closed during reactor power operation and are redundantly demanded to the closed position automatically upon a containment isolation or a containment high radiation signal.

One valve (FCV-1172) is located inside and one valve (FCV-1173) is located outside the containment at each penetration. The Penetration Duct Section between valves is pressurized by air from the Weld Channel and Containment Penetration System whenever they are closed.

On January 5, 1984 at 2:30 PM the WCCPPS supply to Zone 2 (which services the containment ventilation purge penetration) was isolated to replace SOV-1278. After the SOV was tested it was returned to service but the pressurization of the containment penetration for the purge exhaust valves could not be maintained due to excessive leakage. A plant shutdown required by Technical Specifications (3.6.A.5) commenced at 6:30 PM and the unit was removed from service at 9:52 PM.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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FACILITY NAME (1)

INDIAN POINT UNIT 2

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YEAR SEQUENTIAL REVISION

0500024784-001-00013 OF 31

TEXT (If more space is required, use additional NRC Form 366A (17))

The tee-ring seat and compression ring on PCV-1172 were replaced. PCV-1173 which appeared to be slightly open was cycled several times and left in the fully closed position. The penetration was pressurized and valves PCV-1172 and PCV-1110-4 and associated piping joints were tested for leakage with soap solution.

The cause of this event is attributed to a combination of equipment failures: SOV-1278 failing in the closed position and the deterioration of the seat ring elastomers inside containment purge exhaust valve PCV-1172.

The WCCPIS enabled the plant, while under power operation to identify a defect that developed in a containment isolation valve.

The four (4) Containment Purge supply and exhaust valves PCV-1170 thru 1173 are scheduled for Preventive Maintenance during the 1984 Refueling Outage.

The unit was restarted and returned to service at 2:08 PM on Saturday January 7, 1984.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

February 27, 1984

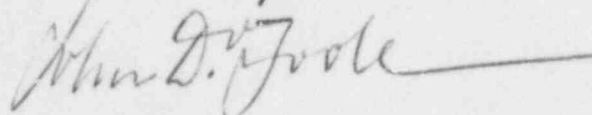
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-84-001-00

Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Dear Dr. Murley:

The attached Licensee Event Report LER-84-001-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Thomas Foley, Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

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