

50-285/76-32

A horizontal number line with arrows at both ends. There are six major tick marks labeled 1, 2, 3, 4, 5, and 6 from left to right. There are also four minor tick marks between each pair of major tick marks, dividing each unit into five equal intervals.

[PLEASE PRINT ALL REQUIRED INFORMATION]

EVENT DESCRIPTION	DATE	TIME	LOCATION	STATUS
1. Registration	10/10/2010	08:00	Room 101	Completed
2. Welcome Address	10/10/2010	08:30	Room 101	Completed
3. Keynote Speech	10/10/2010	09:00	Room 101	Completed
4. Panel Discussion	10/10/2010	09:30	Room 101	Completed
5. Lunch Break	10/10/2010	12:00	Room 101	Completed
6. Workshop	10/10/2010	13:00	Room 101	Completed
7. Q&A Session	10/10/2010	14:00	Room 101	Completed
8. Closing Remarks	10/10/2010	15:00	Room 101	Completed

02 During a routine post shutdown inspection of the containment at 1100 hours on 10-1-76,
7 8 9 80

03 weld leaks were discovered on the controlled bleed-off line on Reactor Coolant Pump
7 8 9 80

04 RC-3C. The leak rate was determined to be less than 0.2 gpm. Controlled bleed-off
7 8 9 80

05 from RC-3C was secured at 0800 on 10-2-76 after RCS cool-down and depressurization had
7 8 9 80

06 been accomplished. (LER 50-285/76-32)
7 8 9 80

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION	
07	C B	E	P	I	P	E	X	X	A	K	O	8	5	N
7	8 9 10	11	12					17	43	44			47	48

[illegible]

08	Weld leaks occurred due to vibration induced fatigue in two locations in the controlled	80
09	bleed-off line. Prior to plant startup the leaking welds will be repaired and the need	80
10	for pipe supports will be evaluated.	80

FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	G	0	0	0	NA		B	Post shutdown inspection-containment		
7	8	9	10	11	12	13	44	45	46	80
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE				
12	M	M	13.84 total curies		RCS leakage to containment building					
7	8	9	10	11	44	45	80			

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
13	0	1	2	E	Total less than .200 man-rem. Maximum individual - 0.022 rem.

PERSONNEL INJURIES

NUMBER				DESCRIPTION	
1	4	0	0	0	NA

OFFSITE CONSEQUENCES

15	NA
7 8 9	80

LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION
16	Z		NA

PUBLICITY

17 NA
7 8 9
8403210152 761012
PDR ADOCK 05000285
S PDR
80

ADDITIONAL FACTORS

18 See Attachments 1, 2 and 3.

19
7 8 9

NAME: W. Dermeyer/R. Andrews

PHONE: 402-426-4011

ATTACHMENT NO. 1

Analysis of Occurrence/Safety Analysis

The reactor coolant system leak experienced was well within Technical Specification 2.1.4 (Reactor Coolant System Leakage Limits). hence, a limiting condition for operation was not violated. Since this failure did result in "abnormal degradation" of the reactor coolant pressure boundary and was not the result of packing or gasket leakage, it has been classified as a reportable occurrence. The leakage experienced was less than 0.2 gpm and did not contribute to any detectable increase in containment activity or dew point. The leakage did result in a spill in the north steam generator cavity and the personnel exposures cited are those attributable to cleanup activities in a radiation area. Since all activity released was confined to the containment structure, the health and safety of the public was not endangered by this occurrence.

ATTACHMENT NO. 2

Corrective Action

Prior to plant startup (after the scheduled refueling outage) the following actions will be completed:

1. The weld failures on RC-3C controlled bleed-off line will be repaired.
2. All reactor coolant pump controlled bleed-off lines will be inspected for deficiencies.
3. The need for additional controlled bleed-off line piping supports will be evaluated (EE/AR 76-50).

ATTACHMENT NO. 3

Failure Data

This is the second instance where weld leakage has occurred on
Reactor Coolant Pump RC-3C controlled bleed-off line.



Omaha Public Power District

1623 HARNEY • OMAHA, NEBRASKA 68102 • TELEPHONE 536-4000 AREA CODE 402

October 12, 1976

FC-295-76

Mr. E. Morris Howard
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive
Suite 1000
Arlington, TX 76012



Dear Mr. Howard:

Reference: Fort Calhoun Station Unit No. 1
Docket No. 50-285

In accordance with the Fort Calhoun Station's Technical Specifications, the Omaha Public Power District, as holder of facility operating license DPR-40, submits three copies of the following licensee event report 50-285/76-32 to satisfy the requirements of Regulatory Guide 1.16.

Sincerely,

W. C. Jones
Section Manager
Operations

WCJ/WDD:rge

Enclosure

cc: Director, Office of Management
Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (3)

10480

Director, Office of Inspection and
Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555 (30)

Mr. L. C. Shalla
SARC Chairman
PRC Chairman
Fort Calhoun File (2)

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