

LICENSEE EVENT REPORT

Update Report
Previous Report DateCONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 12/14/82

0	1	N	C	B	E	P	1	2	0	0	-	0	0	0	0	0	0	0	0	3	4	1	1	1	1	4	5								
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LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE										CAT 58	

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0	1	L	6	0	5	0	-	0	3	2	5	7	1	1	2	0	8	2	8	9									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27									
REPORT SOURCE		DOCKET NUMBER										EVENT DATE										REPORT DATE							

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During steady state plant operation, APRM F was declared inoperable due to an erratic
0 3 | output signal from input LPRM 28-21C. At the time, APRM B was inoperable, as of
0 4 | October 18, 1982, due to erratic outputs from input LPRMs 12-21C and 36-45B. In
0 5 | accordance with technical specifications, a half scram on RPS trip channel B was
0 6 | manually inserted. The failure of these respective LPRMs caused the subject APRMs to
0 7 | have too few inputs. This event did not affect the health and safety of the public.

0 8 | Technical Specifications 3.1.4.3, 3.3.1, 6.9.1.9b

0	9	I	B	11	X	12	Z	13	I	N	S	T	R	U	14	E	15	Z	16		
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26		
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE								COMP. SUBCODE		VALVE SUBCODE					
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.													
18 2		1 1 9		0 3		L		1													
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER					
E 18		Z 19		Z 20		Z 21		0 0 0 0		N 23		Y 24		N 25		G 0 8 0					
33		34		35		36		37		40		41		42		43		44		47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | All LPRMs were tested for APRM operability per MI-16-10 on June 24, 1983. Of the
1 1 | three LPRMs mentioned, only 28-21C did not meet acceptance criteria. However, while
1 2 | performing voltage breakdowns on all LPRMs on August 24, 1983, 28-21C was found to
1 3 | be operable. It is believed that 28-21C had a loose connector that was corrected
1 4 | during testing; therefore, no further action for this event is planned.

1	5	E	28	0	7	2	29	NA	A	31	Operational Event	32
7	8	9	10	11	12	13	14	15	16	17	18	19
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
1 5		0 7 2		NA		A 31		Operational Event				
ACTIVITY		CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE						
1 6		Z 33		NA		NA						
PERSONNEL EXPOSURES		NUMBER		TY. E		DESCRIPTION						
1 7		0 0 0		Z 38		NA						
PERSONNEL INJURIES		NUMBER		DESCRIPTION								
1 8		0 0 0		NA								
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION								
1 9		Z 42		NA								
PUBLICATION		ISSUED		DESCRIPTION								
2 0		N 44		NA								

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PDR ADOCK 05000325
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NRC USE ONLY

NAME OF PREPARER Mary T. Allen

PHONE: 919-457-9521



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
March 14, 1984

FILE: B09-13510C
SERIAL: BSEP/84-0579

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 050-0325
LICENSE NO. DPR-71
SUPPLEMENT TO LICENSEE EVENT REPORT 1-82-119

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MTA/dgr/LETDR1

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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