

**LICENSEE EVENT REPORT**

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
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REPORT SOURCE

L	6	0	5	0	0	0	3	1	5	7	0	7	2	4	8	2	8	0	3	1	3	8	4	9
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60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

A horizontal number line is shown. At the left end, there is a box containing the number 0. To the right of this box, there is another box containing the number 8. The number line continues to the right, ending at a point labeled 100. The line is marked with small vertical tick marks at regular intervals.

0 9  
7 8

SYSTEM CODE  
I E 11  
9 10

CAUSE CODE  
D 12  
11

CAUSE SUBCODE  
X 13  
12

COMP. SUBCODE  
Z 15  
19

VALVE SUBCODE  
Z 16  
20

COMPONENT CODE  
X X X X X X 14  
13 18

SEQUENTIAL REPORT NO.  
0 6 1  
24 26

OCCURRENCE CODE  
0 3  
28 29

REPORT TYPE  
X  
30

REVISION NO.  
1  
32

LER/RO REPORT NUMBER  
17  
21 22

EVENT YEAR  
8 2  
21 22

ACTION TAKEN  
G 18  
33

FUTURE ACTION  
Z 19  
34

EFFECT ON PLANT  
Z 20  
35

SHUTDOWN METHOD  
Z 21  
36

HOURS  
0 0 0 0 22  
37 40

ATTACHMENT SUBMITTED  
Y 23  
41

NPRD-4 FORM SUB.  
N 24  
42

PRIME COMP. SUPPLIER  
L 25  
43

COMPONENT MANUFACTURER  
E 0 7 0 26  
44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 CONTROL ROOM PERSONNEL RESPONSIBLE FOR THE ADMINISTRATIVE SURVEILLANCE OF THE

1 1 ALARM/TRIP SETPOINT COULD NOT DETERMINE THE PROPER SETPOINT. THE SURVEILLANCE

1 2 PROCEDURE WAS REVISED AND SETPOINTS SET AT < 2x BACKGROUND. ADDITIONAL

1 3 ADMINISTRATIVE CONTROLS WITHIN THE OPERATIONS DEPT. OVER PRE-INSTALLATION REVIEW

1 4 AND POST INSTALLATION RELEASE HAVE BEEN INITIATED. SEE ATTACHMENT.

FACILITY STATUS		% POWER			OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1	5	H	28	0 0 0	29 NA	30 B	31 OPERATOR OBSERVATION

ACTIVITY CONTENT  
RELEASED OF RELEASE

1 6 Z (33) Z (34) NA (35)

7 8 9 10 11 44

NA (36)

45 90

PERSONNEL EXPOSURES										
NUMBER			TYPE	DESCRIPTION						
1	7	0	0	0	(37)	Z	(38)	NA		

PERSONNEL INJURIES		80	
NUMBER	DESCRIPTION		
1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100	NA	8403200283	840315

	9	11	12			PDR ADOCK 05000315 S PDR	80
LOSS OF OR DAMAGE TO FACILITY				(43)			
TYPE DESCRIPTION							
1	9	Z	(42)		NA		

8		9		10		80	
PUBLICITY		ISSUED		DESCRIPTION		NRC USE ONLY	
2	0	N	(44)	NA			

NRC USE ONLY

NAME OF PREPARER D. M. DRAPER

PHONE: (616) 465-5901

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10)

ON JULY 24, 1982, WHILE SHUTDOWN FOR REFUELING A NEW TYPE OF CONTAINMENT AIR GASEOUS AND PARTICULATE MONITORS WERE INSTALLED. APPROXIMATELY 37 HOURS LATER ON JULY 26, DURING A DISCUSSION BETWEEN THE DESIGN CHANGE COORDINATOR AND OPERATIONS DEPARTMENT PERSONNEL IT WAS DETERMINED THAT THE ALARM/TRIP SETPOINT VALUE WHICH HAD BEEN SET IN ACCORDANCE WITH APPENDIX "B" T.S. 2.1.3 WAS NOT CONSISTENT WITH THE REQUIREMENTS OF APPENDIX "A" T.S. 3.3.3.1. AT THIS TIME THE ACTION OF 3.9.9 WAS CARRIED OUT.

DURING THIS PERIOD THE REACTOR HEAD WAS IN PLACE AND NO CORE ALTERATIONS TOOK PLACE. NO APPENDIX "B" RELEASE LIMITS WERE EXCEEDED. THIS WAS THE FIRST OCCURRENCE OF THIS TYPE AND NO PROBABLE CONSEQUENCES RESULTED.

27)

CONTROL ROOM PERSONNEL RESPONSIBLE FOR ADMINISTRATIVE SURVEILLANCE OF THE ALARM/TRIP SETPOINT COULD NOT DETERMINE THE PROPER SETPOINT USING THE NORMAL CHANNEL READING BECAUSE IT WAS A NEGATIVE VALUE. THIS RESULTED FROM A BACKGROUND SUBTRACT FEATURE ASSOCIATED WITH THE NEW INSTRUMENTATION. THEY WERE LAX IN REPORTING THEIR DIFFICULTY WITH THE ADMINISTRATIVE SURVEILLANCE PROCEDURE TO THE PROPER PERSONNEL.

THE BACKGROUND SUBTRACT WAS DISABLED, THE ALARM SETPOINT SET AT  $\leq 2 \times$  BACKGROUND AND THE SURVEILLANCE PROCEDURE REVISED.

THE PLANT MANAGERS INSTRUCTION RELATING TO DESIGN CHANGES HAS BEEN REVISED TO INCLUDE A CONDITIONAL RELEASE WHICH MUST BE SIGNED BY THE SHIFT SUPERVISOR BEFORE THE EQUIPMENT BE CONSIDERED OPERATIONAL, THIS PROVIDES AN ADDITIONAL MANAGEMENT STAFF REVIEW OF THE CHANGE.



**INDIANA & MICHIGAN ELECTRIC COMPANY**

DONALD C. COOK NUCLEAR PLANT  
P.O. Box 458, Bridgman, Michigan 49106  
(616) 465-5901

March 13, 1984

Mr. J.G. Keppler, Regional Administrator  
United States Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Operating License DPR-58  
Docket No. 50-315

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical  
Specifications, the following report/s are submitted:

RO 82-061/03X-1

Sincerely,

for W.G. Smith, Jr.  
Plant Manager

/cbm

cc: John E. Dolan  
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