

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 0 1 1										PAGE (3) 1 OF 0 2				
TITLE (4) Diesel Generator No. 4 Auto Start Due to Loss of Plant Emergency AC Bus E-4																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)									
0	2	1	4	8	4	8	4	0	0	3	0	0	0	3	1	5	8	4	0 5 0 0 0 1 3 2 1 5					
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 9 5			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
			20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)									
			20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
			20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																								
NAME M. J. Pastva, Jr., Regulatory Technician												TELEPHONE NUMBER 9 1 9 4 5 7 - 9 5 2 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)				MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing a functional test of degraded voltage instrumentation for the 4160V emergency buses concurrent with an operability test of alarm and logic instrumentation for plant emergency diesel generators (DG), the normal power supply feeder to emergency bus E-4 automatically opened due to a bus undervoltage condition as sensed by the bus degraded voltage relay, 27 DV. Group 3 and Group 6 isolations occurred. Within 10 seconds DG No. 4 automatically started and reenergized E-4. At the time of this event, Units 1 and 2 were at power operation of 99 percent and 95 percent, respectively. Within 22 minutes of the event, the normal supply feeder breaker to E-4 was reclosed. The cause of this event could not be determined. An operability check of 27 DV showed the device is able to perform its designed function. This event did not affect the health and safety of the public. No further action is planned regarding this event.

The functional test being performed, PT-12.7.3P, is a monthly test performed in the degraded voltage relays for each emergency bus. This procedure does not have a history of problems and has been reviewed for technical adequacy; therefore, it has been determined that the procedure is adequate. Additionally, no identified deficiencies existed in test performance to contribute to the event. This portion of the test was subsequently reperfomed satisfactorily.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 4 8 4 - 0 0 3 - C 0 0 2 OF 0 2	LER NUMBER (9)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On February 14, 1984 at 1307, during the performance of a functional test of degraded voltage instrumentation for the 4160V emergency buses concurrent with an operability test of alarm and logic instrumentation for plant emergency diesel generators (DG), the normal power supply feeder to emergency bus E-4 automatically opened due to a bus undervoltage condition as sensed by the bus degraded voltage relay, 27 DV. The deenergization of E-4 resulted in a loss of power supply to the following Unit 1 and 2 E-4 loads:

Unit 1

Conventional Service Water (CSW)
Pump Motor 1A
Residual Heat Removal (RHR)
Pump Motor 1B
RHR Service Water Pump Motor 1B
*

Unit 2

CSW Pump Motor 2B
Nuclear Service Water
Pump Motor 2B
Control Rod Drive Pump Motor 2B
Reactor Core Spray Pump Motor 2B
RHR Service Water Pump Motor 2B
RHR Pump Motor 2B
*

Note: *The alternate power supply to the plant Fire Protection System pump motor was also due to the loss of E-4.

As per design, within ten seconds DG No. 4 automatically started and reenergized E-4. The deenergization of E-4 resulted in Group 3 and 6 primary containment isolations on Unit 2. These isolations were verified by the Control Operator observing the position indication lights of the subject Group 3 and 6 valves. These group isolations placed the unit in the most conservative mode. At the time of this event, Units 1 and 2 were at power operation of 99 percent and 95 percent, respectively; therefore, it is felt that the event occurred in the most limiting situation.

Within 22 minutes of the event, the normal supply feeder breaker to E-4 was reclosed. The cause of the event could not be determined. An operability check of 27 DV showed the subject device is capable of performing its designed function. No further action is planned regarding this event.



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
March 15, 1984

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SERIAL: BSEP/84-0544

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-84-3

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/jlh/LETJH1

Enclosure

cc: Mr. J. P. O'Reilly

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