

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Project - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7				PAGE(S) 1 OF 0 2								
TITLE (4) Unplanned RPS Trip During Operational Leakage Test																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
0	2	1	9	8	4	8	4	0	1	4	0	0	3	1	4	8	4	0	5	0	0	0
OPERATING MODE (9) 4			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																			
POWER LEVEL (10) 2.23			20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
			20.406(a)(1)(i)				50.38(a)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)							
			20.406(a)(1)(ii)				50.38(a)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 388A)							
			20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)											
			20.406(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)											
			20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																						
NAME L. D. Kassakatis, Plant Compliance Engineer										TELEPHONE NUMBER AREA CODE 5 0 9 3 7 7 - 2 5 0 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 4/27																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC												
D	S	B		No																		
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)																<input checked="" type="checkbox"/> NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing an operational leakage test of the Reactor Pressure Vessel (RPV) an unplanned Reactor Protection System (RPS) trip occurred, when the water rejection flow path from the RPV was isolated without isolating or reducing the amount of water being injected into the RPV. This condition was reported to the NRC pursuant to 10 CFR 50.72(b)(2)(ii). This LER provides written follow-up pursuant to 10 CFR 50.73(a)(2)(iv).

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Project - Unit 2	0500039784	—	014	—	0002	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

## Plant Operating Conditions - Prior to and During the Event:

- a) Power Level - Zero
- b) Mode Shutdown
- c) Prior to Initial Power Operations

After completing rework on an IRM-E flange, the RPV pressure was being increased to 1000 PSIG for an operational leakage test. RPV temperatures were >125°F. At the same time preparations were being made for an outage of 4.16KV Bus SM-7 to complete Plant modifications and maintenance. When RPS-Bus A was transferred to its alternate power source, because this is a dead bus transfer a momentary loss of power was experienced. The loss of power initiated a NS<sup>4</sup> isolation signal causing RWCU-V-4 to close. RPV pressure was being controlled by water injection via the Control Rod Drive System and water rejection via RWCU. When the RWCU valve closed, the rejection path was eliminated and RPV pressure increased causing an unplanned RPS trip. The pressure increase was sufficient to cause the RPS trip on Hi Reactor Pressure (1037 PSIG). However, Reactor Pressure did not reach the Tech Spec Limit of 1050 PSIG steam dome pressure. Since the power loss was only momentary RWCU-V-4 was reopened, the RPS trip reset, and RPV pressure returned to 1000 PSIG to allow completion of the leakage test.

## Corrective Action:

The Plant configuration was not compatible with the bus outage work. In the future with the Plant in a similar configuration, the power to RPS-Buses "A" and/or "B" will not be transferred or interrupted intentionally unless procedural steps are taken to prevent an inadvertent RPS trip initiation or NS<sup>4</sup> initiation.

## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397  
March 14, 1984

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Washington, D.C. 20555

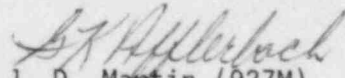
Subject: **NUCLEAR PROJECT NO. 2**  
**LICENSEE EVENT REPORT NO. 84-014**

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-014 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of noncompliance, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 0550 hours on February 19, 1984.

Very truly yours,

  
J. D. Martin (927M)  
WNP-2 Plant Manager

JDM:de

Enclosure:  
Licensee Event Report No. 84-014

cc: Mr. John B. Martin, Administrator  
Region V, Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
1450 Maria Lane  
Walnut Creek, California 94596  
Mr. A. D. Toth, NRC Resident Inspector (901A)

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