

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2)	PAGE (3)						
			0	5	0	0	0	3	1

TITLE (4) DEFECTIVE REACTOR WATER LEVEL SWITCH

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)															
0	2	1	3	8	4	8	4	—	0	0	6	—	0	0	0	3	1	4	8	4									
																		0	5	0	0	0							

OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)						
POWER LEVEL (10)	100		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
			20.405(a)(1)(i)		50.36(c)(1)	<input checked="" type="checkbox"/>	50.73(a)(2)(v)		73.71(c)
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)					
NAME	TELEPHONE NUMBER				
Hartford N. Keith	<table border="1"> <tr> <td>AREA CODE</td> <td></td> </tr> <tr> <td>3, 1, 5</td> <td>3, 4, 2, -, 3, 8, 4, 0</td> </tr> </table>	AREA CODE		3, 1, 5	3, 4, 2, -, 3, 8, 4, 0
AREA CODE					
3, 1, 5	3, 4, 2, -, 3, 8, 4, 0				

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	J	E L I S	I 2 Q 4	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1300 spaces, i.e., approximately fifteen single-space typewritten lines) (15)

During normal full power operation on February 13 and March 2, 1984 Reactor Water Level Indicating Switch 02-3-LIS-101D was found out of Technical Specification Limit. The instrument is required to be set at or less than 222.5 inches above Top of Active Fuel (TAF). On February 13, 1984 it was found set at 222.8 inches above TAF and on March 2, 1984 it was found set at 228.0 inches above TAF. A review of past performance revealed erratic behavior since installation in December, 1982.

Evaluation of the installed instrument and an identical instrument drawn from spare parts indicates that tightening of the "switch lock" causes bending of switch backing plates resulting in possible excessive friction between the switch actuator cam and actuator arm on both instruments.

Immediate corrective action is to perform surveillance of the instrument at an increased frequency until replacement can be completed. The instrument drawn from spare parts has been sent to the manufacturer with a request to evaluate the problem and repair or replace the instrument.

Evaluation of surveillance/calibration data for other instruments performing the same safety function did not indicate a similar problem. Long term corrective action includes replacement of the instrument with an Analog Trip Transmitter Unit.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

FACILITY NAME (1) JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3	LER NUMBER (6)			PAGE (3)		
		YEAR 8 4	SEQUENTIAL NUMBER 0 0 6	REVISION NUMBER 0 0		OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal full power operation while conducting surveillance Reactor Water Level Indicating Switch 02-3-LIS-101D was found out of Technical Specification Limit. The instrument set point as found was 222.8 inches above Top Of Active Fuel (TAF) when required by Technical Specification Table 3.2-2 to be set at or less than 222.5 inches above TAF. The switch was immediately adjusted to within Technical Specifications and procedural tolerance and placed on once per week increased surveillance frequency. The cause of the out of tolerance data was believed to be instrument drift.

Two (2) weeks later on March 2, 1984, instrument 02-3-LIS-101D was again found out of Technical Specification limits. The switch was found at 228.0 inches above TAF and was immediately adjusted to within Technical Specification and procedural tolerance.

Investigation into the past performance of Instrument 02-3-LIS-101D revealed that the currently installed instrument was installed on December 21, 1982. Review of surveillance and calibration data from December, 1982 to present indicates the following.

- Feb. 1, 1983 Out of Technical Specification Limit. Adjusted and increased surveillance to weekly. Submitted LER 83-007/03L-0.
- Feb. 28, 1983 Approaching procedural guide value in direction of Technical Specification Limit. Adjusted instrument and continued weekly surveillance.
- March 28, 1983 Discontinued weekly surveillance.
- April 28, 1983 Approaching procedural guide value in direction of Technical Specification Limit. Adjusted instrument.
- May 23, 1983 Found set point below procedural guide limit (set point moved away from Technical Specification Limit). Did not adjust instrument.
- Aug. 25, 1983 Found set point at approximately same point as set point was left on May 23, 1983. Adjusted to within procedural guide value.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

(PIRES: 8/31/85)

FACILITY NAME (1) JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			- 0 0 6	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Sept. 7, 1983 Found set above procedural guide value but within Technical Specification Limit. Adjusted to within guide value.

Dec. 20, 1983 Found above Technical Specification Limit. Adjusted and increased surveillance to weekly. Submitted LER 83-064/03L-0.

Jan. 11, 1984 Discontinued weekly surveillance.

Jan. 17, 1984 Found approaching Technical Specification Limit. Adjusted instrument.

Feb. 13, 1984 Found above Technical Specification Limit. Adjusted and increased surveillance to weekly.

March 2, 1984 Found above Technical Specification Limit. Adjusted instrument and initiated plans for replacement.

March 2, 1984 Plant Shutdown for maintenance and modification.

March 13, 1984 Found within procedural guide value. Plant start-up initiated.

During the plant shutdown from March 2, 1984 to March 13, 1984 investigation of the cause of the erratic behavior of the installed instrument was conducted. Based on this investigation it appears that tightening the "switch lock" bends the switch backing plates causing excessive friction between the actuator cam and actuator arm. Evaluation of an identical instrument from spare parts revealed a similar problem. The instrument manufacturer (ITT Barton) has been requested to evaluate the instrument which had been drawn from spares and repair or provide a replacement.

Review of surveillance and calibration data for instruments 02-3-LIS-101A,B, and C (which perform the same safety function) does not indicate that the problem exists with these other instruments.

Until a new or repaired instrument is received from the manufacturer, surveillance of the installed instrument will be conducted at an increased surveillance frequency.

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Lycoming, New York 13093
315 342.3840



March 14, 1984
JAFP-84-0296

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

REFERENCE: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 84-006-00

Dear Sir:

We have enclosed the referenced Licensee Event Report
in accordance with 10CFR50.73.

If there are any questions concerning this report,
please contact Mr. Hartford N. Keith at (315) 342-3840,
Extension 230.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Corbin A. McNeill, Jr.', followed by a horizontal line.

CORBIN A. McNEILL, JR.
RESIDENT MANAGER

CAM/HNK/jmk
Enclosure

CC: USNRC, Region I (1)
INPO Records Center, Atlanta, Georgia (1)
Internal Power Authority Distribution
NRC Resident Inspector
Document Control Center
LER/OR File

IE22
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