

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-205

UNIT Fort Calhoun Station

DATE March 13, 1984

COMPLETED BY T. P. Matthews

TELEPHONE (402) 536-4733

MONTH February, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>454.7</u>
2	<u>454.7</u>
3	<u>454.5</u>
4	<u>455.2</u>
5	<u>454.9</u>
6	<u>454.6</u>
7	<u>454.7</u>
8	<u>454.9</u>
9	<u>454.4</u>
10	<u>454.3</u>
11	<u>454.6</u>
12	<u>454.4</u>
13	<u>454.7</u>
14	<u>454.5</u>
15	<u>454.0</u>
16	<u>453.9</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>453.6</u>
18	<u>443.5</u>
19	<u>437.7</u>
20	<u>430.5</u>
21	<u>425.1</u>
22	<u>419.4</u>
23	<u>413.6</u>
24	<u>407.7</u>
25	<u>402.7</u>
26	<u>397.4</u>
27	<u>392.3</u>
28	<u>387.1</u>
29	<u>382.0</u>
30	<u> </u>
31	<u> </u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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PDR ADOCK 05000285
R PDR

IE 24
11

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE March 13, 1984
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: February, 1984
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 461
7. Maximum Dependable Capacity (Net MWe): 438
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	696.0	1,440.0	91,442.0
12. Number Of Hours Reactor Was Critical	696.0	1,440.0	71,333.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.0
14. Hours Generator On-Line	696.0	1,440.0	70,842.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	997,882.7	2,103,770.5	88,863,484.2
17. Gross Electrical Energy Generated (MWH)	319,880.0	675,442.0	28,993,011.0
18. Net Electrical Energy Generated (MWH)	303,974.7	642,754.7	27,722,623.4
19. Unit Service Factor	100.0	100.0	77.5
20. Unit Availability Factor	100.0	100.0	77.5
21. Unit Capacity Factor (Using MDC Net)	99.7	101.9	66.1
22. Unit Capacity Factor (Using DER Net)	91.4	93.4	63.7
23. Unit Forced Outage Rate	0.0	0.0	3.5
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>1984 refueling outage scheduled to start March 3, 1984</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
 26. Units In Test Status (Prior to Commercial Operation):
- | | Forecast | Achieved |
|----------------------|----------|----------|
| INITIAL CRITICALITY | _____ | _____ |
| INITIAL ELECTRICITY | _____ | _____ |
| COMMERCIAL OPERATION | _____ | _____ |

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1984

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE March 13, 1984
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									No unit shutdowns during the month of February, 1984.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending February 1984.

1. Scheduled date for next refueling shutdown. March 1984
2. Scheduled date for restart following refueling. May 1984
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

- a. If answer is yes, what, in general, will these be?

Technical Specification changes include the incorporation of the statistical combination of uncertainties program, higher radial peaking factors, a different TM/LP equation, a more negative moderator temperature coefficient of reactivity limit, more restrictive LHR and DNBR LCO's on axial shape index, and the addition of asymmetric steam generator transient protection trip function.

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

- c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information.

Tech. Specs. have been submitted

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
- | | | |
|---|------------|------------|
| a) in the core | <u>133</u> | assemblies |
| b) in the spent fuel pool | <u>265</u> | " |
| c) spent fuel pool storage capacity | <u>729</u> | " |
| d) planned spent fuel pool storage capacity | <u>*</u> | |

*may be increased via fuel pin consolidation

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1996

Prepared by

JK Gayer

Date

March 1, 1984

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

February, 1984
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station continued its record online run at 100% power until February 18, 1984, when it began an end of cycle coast down at about 1% per day.

Preparations continue for the 1984 refueling and maintenance outage which is scheduled to begin March 3.

New fuel receipt for Cycle 9 has been completed.

The Nuclear Regulatory Commission was onsite during February to audit the training department.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
84-001	Auxiliary Building Crane Interlocks Bypassed.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel Assembly Uplift Condition Detection.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

<u>Procedure</u>	<u>Description</u>
SP-WDS-16	Spent Resin Disposal. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure only provides for sampling of a waste gas decay tank. The sample is returned to the waste disposal system, so there is no gas release.
SP-VA-80	Hydrogen Purge System Test. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure only checks operability of fans and cleanliness of the filters.

System Acceptance Committee Packages for February, 1984:

<u>Package</u>	<u>Description/Analysis</u>
DCR 74B-18	Loose Parts Monitoring System. This modification added sensors and a system to monitor vibration and thus detect loose parts in vital systems. This improves the detection of potentially damaging vibration. This modification has no adverse effect on the safety analysis.
DCR 74A-77	Refueling Machine Grapple and Spreader Modification. This modification installed an articulated grapple and appropriate spreader that was consistent with the modified fuel. This made it easier, and thus safer, to handle the fuel bundles. This modification has no adverse effect on the safety analysis.
DCR 77-01	Isolation Valves for Diesel Generator Air Compressors. This modification installed a valve in the diesel generator air compressor lines so that one could isolate the air receivers, and not have to blow down the tank during maintenance. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

System Acceptance Committee Packages for February, 1984:

<u>Package</u>	<u>Description/Analysis</u>
DCR 77-59	Radioactive Waste System Modification. This modification installed piping to direct waste from the evaporator to the floor drain in the room. This modification has no adverse effect on the safety analysis.
EEAR FC-80-65	Electrical Power for Emergency Communication. This modification is not safety related, therefore, there is no adverse effect on the safety analysis.
EEAR FC-80-23	Replacement of Alarm Typer on P-250. This modification replaced a slower printer with one that is much faster, and thus does not cause information to be lost. This modification has no adverse effect on the safety analysis.
EEAR FC-83-31	HCV-348 Shaft Change. This modification changed the shaft and gearbox and resulted in increased speed of operation. This modification has no adverse effect on the safety analysis.
EEAR FC-81-81	AI-41A/B Bus Failure. This modification installed bus failure annunciation only. Annunciation has no effect on the safety analysis.
EEAR FC-79-215	Additional Gai-tronics in Containment. The gai-tronics system is not safety related, therefore, there is no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(continued)

System Acceptance Committee Packages for February, 1984:

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-80-110	Diesel Generator Tachometer Circuit Noise. This modification eliminates A-C noise associated with the diesel generator speed sensing circuitry. Installation or removal of this capacitor will not effect the ability of the diesel generator from starting and operating under accident circumstances. Installation of this capacitor will help circuit stability and help control the sensing circuitry setpoints from drifting. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

The Fort Calhoun Station is currently performing B and C penetration tests. A report will be sent out at the end of the refueling outage.

F. CHANGES IN PLANT OPERATING STAFF

Mr. Gene Creamer started March 6, 1984, as an Auxiliary Operator - Nuclear.

G. TRAINING

Operations personnel completed ERF computer training. General employee training classes were increased to accomodate the additional personnel that will be supporting the 1984 outage. Other training continued as scheduled.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

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Page Five
February, 1984

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

March 13, 1984
LIC-84-074

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

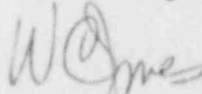
Reference: Docket No. 50-285

Dear Mr. DeYoung:

February Monthly
Operating Report

Please find enclosed ten (10) copies of the February Monthly
Operating Report for the Port Calhoun Station Unit No. 1.

Sincerely,



W. C. Jones
Division Manager
Production Operations

WCJ/TPM:jmm

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File