NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Peach Bottom Atomic Power Station - Unit 2	0 5 0 0 0 2 7 7	1 OF 0 3

TITLE (4)

Packing leak on the RHR full-flow test line isolation valve (MO2-10-34B)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	1 3	8 4	8 4	- 0 0 4	- 0 0	0 3	1 4	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 9 9		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
B. L. Clark, Senior Engineer - Special Projects	AREA CODE 2 1 5 8 4 1 - 5 0 1 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	B O	I S V	W 0 3 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)				
X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract 2-84-04

On February 13, 1984, while performing Local Leak Rate Tests on the 'B' RHR system valves on Unit 2, a packing leak was discovered using leak detection fluid on the RHR full-flow test line isolation valve, MO2-10-34B, which allowed venting of torus atmosphere to secondary containment. The leakage was non-isolatable from primary containment except by backseating the valve. The leakage could not be to be measured due to the piping configuration. The cause was a result of loose packing. The packing was tightened, the valve was stroke tested satisfactorily and returned to service within 3 hours from the time of its discovery.

This LER is submitted pursuant to the fact that a determination could not be made as to whether or not the containment leakage was within allowable Technical Specification limits. The applicable Tech. Spec. is 4.7.A.2.d.

Description of the Event:

On February 13, 1984, Unit 2 was at 99% power level. At 4:30 p.m., while performing a Local Leak Rate Test (LLRT) on the 2B Residual Heat Removal (RHR) system valves, a decay in the test pressure indicated that leakage existed. Investigation using leak detection fluid revealed a packing leak on the full-flow test line isolation valve, MO2-10-34B. However, when the LLRT pressure was removed from the valve, it was noted that leakage continued through the valve packing. It was determined that this leakage was from the containment side of the valve and was therefore venting the torus atmosphere to secondary containment. The leakage was not isolatable from primary containment except by backseating of the valve. Also, the leakage could not be measured due to the piping configuration. This LER is submitted pursuant to the fact that a determination could not be made as to whether or not the containment leakage was within allowable Technical Specification limits. The applicable Tech. Spec. is 4.7.A.2.d.

Consequences of the Event:

The leakage was minimal due to the small clearances between the stem and the packing. Likewise, the leakage was contained in secondary containment.

Cause of the Event:

Cause of the event was due to loose packing of the MO2-10-34B valve (Walworth, Model 5281 WE).

Corrective Actions:

The packing on the MO2-10-34B valve was tightened with no leakage verified using leak detection fluid. The valve was stroke tested satisfactorily and returned to service by 7:30 p.m. on February 13, 1984. The packings of the similar valve, MO2-10-34A, on Unit 2 and also the MO3-10-34A and B valves on Unit 3 were checked for leakage using leak detection fluid. No leakage was identified. Likewise, a similar piping configuration was identified on the RHR torus spray isolation valves, MO-10-38A and B, on Units 2 and 3. The packings of these valves were also checked for leakage using leak detection fluid and no leakage was identified.

The routine LLRT checks the valve seat leaktightness in which case the MO2-10-34B valve passed. However, the current piping configuration does not permit testing the valve bonnet and stem packing leaktightness because it communicates directly to the torus air space. Therefore, maintenance procedures are being revised to require a lantern ring to be installed in the stem packing of the MO-10-34A and B and the MO-10-38A and B valves on both Unit 2 and 3, any time the valves are repacked. This will facilitate local leak rate testing of the packing. Additionally, the maintenance procedures will require that an LLRT be performed on the packing any time these valves are repacked. The initial installation of the lantern ring setup in the packing of the MO-10-34A and B and the MO-10-38A and B valves on both Unit 2 and 3 is expected to be accomplished during the next refueling outage for each unit.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

March 14, 1984

Docket No. 50-277

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report

This LER deals with a packing leak on the 'B' RHR full flow test line isolation valve, MO2-10-34B, on Unit 2.

Reference:	Docket No. 50-277
Report Number:	2-84-04
Revision Number:	00
Event Date:	February 13, 1984
Report Date:	March 14, 1984
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the fact that a determination could not be made as to whether or not the containment leakage was within allowable Technical Specification limits. The applicable Tech. Spec. is 4.7.A.2.d.

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

Mr. A. R. Blough
Site Inspector

IE22
1/1