



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

August 10, 1995
BECO Ltr. #95-081

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

JULY 1995 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


L.J. Olivier

WJM/laa/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE 8/10/95
 COMPLETED BY: W.J. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

1. Unit Name Pilgrim I
2. Reporting Period July 1995
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744.0</u>	<u>5087.0</u>	<u>198479.0</u>
12. Hours Reactor Critical	<u>744.0</u>	<u>3393.0</u>	<u>122593.1</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>3289.8</u>	<u>118150.9</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1439150.0</u>	<u>6319382.0</u>	<u>209074598.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>495770.0</u>	<u>2172570.0</u>	<u>70792244.0</u>
18. Net Electrical Energy Generated(MWH)	<u>476983.0</u>	<u>2089479.0</u>	<u>68047256.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>64.7</u>	<u>59.5</u>
20. Unit Availability Factor	<u>100.0</u>	<u>64.7</u>	<u>59.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>95.7</u>	<u>61.3</u>	<u>51.2</u>
22. Unit Capacity Factor (Using DER Net)	<u>97.9</u>	<u>62.7</u>	<u>52.3</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>7.3</u>	<u>12.3</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each)	<u>NONE</u>		
25. If Shutdown at End of Report Period, Estimated Date of Startup - Unit Operating			

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
DATE: 8/10/95
COMPLETED BY: W.J. Munro
TELEPHONE: (508) 830-8474

MONTH July 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	666	17	664
2	666	18	663
3	664	19	417
4	664	20	338
5	664	21	502
6	666	22	664
7	667	23	664
8	667	24	664
9	666	25	665
10	666	26	665
11	667	27	665
12	666	28	664
13	667	29	664
14	668	30	661
15	667	31	661
16	663		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR JULY 1995

The unit started the reporting period at approximately 100 percent core thermal power (CTP), and essentially maintained this power level until July 19, 1995. At 0635 hours on July 19, 1995 the "B" seawater pump was secured due to a loss of lubricating oil to the lower motor bearing. Reactor power was reduced to approximately 60 percent CTP. Following repair and replacement of the motor lower bearing, the "B" seawater pump was returned to service on July 21, 1995 at 0556 hours. Reactor power was increased and 100 percent power was achieved at 2040 hours on July 21, 1995 where it was essentially maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF JULY 1995

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: March 29, 1997.
3. Scheduled date for restart following next refueling: May 12, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1765 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

DOCKET NO: 50-293

NAME: Pilgrim I

DATE: 8/10/95

COMPLETED BY: W.J. Munro

TELEPHONE: (508) 830-8474

REPORT MONTH: July 1995

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Sampling System	Inboard Primary Containment Isolation System (PCIS) valve AO-220-44	Valve would not close from C904 control switch during performance of Surveillance 8.7.4.3	To be determined	Outboard PCIS AO-220-45 valve was deactivated in the closed position.	AO-220-44 will be examined at next outage.	N/A
Core Spray System	Core Spray Outboard Injection Valve MO-1400-24B control switch 14A-52B. (Part # GE 10BB161)	Due to significant degradation of the SBM control switch for MO-1400-24B, Core Spray "B" Loop was taken out of service for replacement of the control switch.	Degraded cam followers in the SBM control switch.	SBM Control Switch 14A-52B for MO-1400-24B valve was replaced with an upgraded GE SBM control switch (Part #GE 213A6944ABP037)	Replace all SBM control switches used in safety related applications with upgraded version.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-293

NAME: Pilgrim IDATE: 8/10/95COMPLETED BY: W.J. MunroTELEPHONE: (508) 830-8474REPORT MONTH: July 1995

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
06	07/19/95	F	0.0	B	N/A	N/A	N/A	N/A	Power reduction due to "B" seawater pump motor bearing failure

<p>1</p> <p>F-Forced S-Sched</p>	<p>2</p> <p>A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other</p>	<p>3</p> <p>1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other</p>	<p>4&5</p> <p>Exhibit F & H instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)</p>
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