NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	1 OF 0 2

TITLE (4)

Unit 1 Scram From High High-Pressure Turbine First Stage Pressure

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
												0 5 0 0 0
0 2	2 2	8 4	8 4	—	0 1 4	—	0 0	0 3		8 4		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 1 7		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
C. J. Rozear	AREA CODE 2 0 5 7 2 9 - 0 7 8 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During startup of unit 1, while warming the turbine, the reactor scrammed when the turbine high-pressure first stage pressure exceeded 142 psig with the turbine stop valves closed. All redundant systems were operable. The cause of the scram is considered to be procedural error. The operating instruction will be revised to keep the actual turbine first stage pressure from exceeding 135 psig during turbine stop valve closure.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	8 4	— 0 1 4	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On February 22, 1984, with unit 1 operating at 7 percent power, unit 2 in startup and unit 3 in a refueling outage, the unit 1 reactor scrammed during startup when the turbine high-pressure first stage pressure exceeded 142 psig with the turbine stop valve (SHV) closed. The A2 and B1 reactor trip actuators were picked up and initiated the scram, indicating that only the setpoints of PT-1-81B and -19A (turbine first stage pressure permissive) (JC) were reached. Subsequent review indicates that the turbine first stage pressure permissive setpoint is reduced 12 psig for instrument accuracy and the actual setpoint is 142 psig vs. 154 psig specified in the Technical Specifications.

The cause of this scram is considered to be procedural inadequacy. Operating instruction OI-47 will be revised to keep the actual turbine first stage pressure from exceeding 135 psig during turbine stop valve closure.

This scram was normal and had no safety significance. There was no operation of the high pressure coolant injection system (BJ), the reactor core isolation cooling system (BH), main steam isolation valves (ISV) or main steam relief valves (RV).

Responsible Plant Section - OP

Previous Similar Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

MAR 13 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFRO-50-259/84014

The enclosed report provides details concerning unit 1 scram from high high-pressure turbine first stage pressure. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

for C E Sorrell
G. T. Jones
Power Plant Superintendent
Browns Ferry Nuclear Plant

Enclosure
cc (Enclosure):
Regional Administrator
U. S Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, GA 30303

NRC Inspector, Browns Ferry Nuclear Plant

IE-22
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