

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

FEBRUARY 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit commenced commercial generation of power on January 1, 1984.

This report was compiled by Diane L. Lin, telephone number (815)357-6761, extension 499.

## II. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

FEBRUARY 1-3 The unit started the reporting period at approximately 87% power. On February 3 at 1345 hours the reactor scrammed due to an inadvertent initiation of RCIC during the performance of LIS-NB-04, Reactor Vessel Low Low Water Level RCIC Initiation, Low-Low-Low Water Level LPCS/RHR Initiation, and ADS Permissive Calibration and Functional Test. The reactor was critical for 61 hours and 45 minutes.

FEBRUARY 4-29 The reactor went critical at 0450 hours on February 4. At 1108 hours on February 4 the main generator was synchronized to the grid and loaded. At 1145 hours on February 4 reactor power was 25%. At 0740 hours on February 5 reactor power was 66%. At 0645 hours on February 9 reactor power was 86%. On February 13 at 0044 hours the reactor scrammed due to the 'A' condenser boot seal which failed and subsequent loss of condenser vacuum. The reactor was critical for 211 hours and 54 minutes.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

A. Amendments to Facility License or Technical Specifications.

There were no amendments to the facility license or Technical Specifications during the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

## ATTACHMENT A

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENTLTP-300-7  
Revision 3  
March 1, 1983  
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L25546		Valve 1B33-F340A	Open limitswitch not making	Open indicating light does not work	Set open limitswitch
L29956		'C' RHR Full Flow Test to Suppression Pool Valve.	Bad packing	Valve was leaking	Repacked valve.
L30130		Computer Room Temperature Indicator	Indicator hanging from wires.	Upscale indication "pegged high."	Placed indicator back in panel.
L32201		'B' IRM	Bad drive motor, flexible shaft and gear box.	IRM will not withdraw.	Replaced drive motor, flexible shaft and gear box.
L32529		Standby Gas Treatment WRGM Detection Skid.	Bad pump diaphragm	Skid leaks.	Replaced pump diaphragm.
L32559		'S' ADS Valve Accumulator Low Pressure Switch.	Switch out-of-calibration	Hathaway alarm is up, ESF alarm is not up.	Recalibrated switch.

## ATTACHMENT A Cont'd

LTP-300-7  
Revision 3  
March 1, 1983  
5CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L32624		LPCI 'A' Pump Suction Pres- sure Gauge.	Pressure indicator out-of-adjustment.	Gauge reads 23 psig, should read ~10psig.	Rezeroed pressure indicator.
L32661		Recorder 1TR -CM038	Pivot on alarm switch mechanism was impeding movement.	Recorder sticks on point 4.	Oiled pivot.
L32741		Suppression Pool Water RTD	Out-of-calibration	A and B channel read differently.	Performed calibration
L32783		HPCS Water Leg Suction Gauge.	Out-of-calibration	Gauge reads high	Recalibrated gauge.
L32784		HPCS Suction Low Pressure Gauge.	Out-of-calibration	Gauge reads 0# should be 15-25#	Recalibrated gauge.
L33091		Scram Isolation Drain Valve.	Limit switch out-of- adjustment	Dual indication when valve is closed.	Adjusted limit switch



ATTACHMENT A (CONT'D)  
CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

LTP-300-7  
Revision 3  
March 1, 1983  
5

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L33197		Valve 1E32-F007	Bad torque switch	Valve would not open automatically.	Replaced torque switch.
L33250		RHR 'B' Testable check bypass valve	Contact screw on torque switch loose	Valve indicates full closed out can be manually closed further.	Tightened contact screw.
L33255		'A' LPCI Testable Check Valve.	Limit switch not made in the closed position.	Limit switch doesn't pick up when valve is full closed.	Adjusted limit switch
L33256		'C' LPCI Testable Check Valve.	Bad air cylinder	Air cylinder is blowing air.	Replaced air cylinder.



#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, February 1 through February 29, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-002-00	1/6/84	Loss Feed Pump Causing Scram
84-003-00	1/13/84	Reactor Scram-Main Generator Trip.
84-004-00	1/18/84	Secondary Containment Isolation
84-005-00	1/16/84	Scram
84-006-00	1/21/84	B&F APRM's Reading Non-conservative.
84-007-00	2/3/84	Unit 1 Reactor Scram.

## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

## OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LaSalle One  
DATE March 9, 1984  
COMPLETED BY Diane L. Lin  
TELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: February 1984 GROSS HOURS IN REPORTING PERIOD: 696
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): \_\_\_\_\_
4. REASONS FOR RESTRICTION (IF ANY): \_\_\_\_\_

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>273.7</u>	<u>817.7</u>	<u>817.7</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>407.3</u>	<u>589.4</u>	<u>589.4</u>
7. HOURS GENERATOR ON LINE	<u>267.4</u>	<u>734.9</u>	<u>734.9</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>7040020</u>	<u>1803672</u>	<u>1803672</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>229288</u>	<u>576910</u>	<u>576910</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>211758</u>	<u>539632</u>	<u>539632</u>
12. REACTOR SERVICE FACTOR	<u>39.3%</u>	<u>56.8%</u>	<u>56.8%</u>
13. REACTOR AVAILABILITY FACTOR	<u>97.8%</u>	<u>97.7%</u>	<u>97.7%</u>
14. UNIT SERVICE FACTOR	<u>38.4%</u>	<u>51.0%</u>	<u>51.0%</u>
15. UNIT AVAILABILITY FACTOR	<u>38.4%</u>	<u>51.1%</u>	<u>51.1%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>29.4%</u>	<u>36.2%</u>	<u>36.2%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>28.2%</u>	<u>34.8%</u>	<u>34.8%</u>
18. UNIT FORCED OUTAGE RATE	<u>61.6%</u>	<u>44.8%</u>	<u>44.8%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>3/6/84</u>			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			

	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u>        </u>	<u>6/21/82</u>
INITIAL ELECTRICITY	<u>        </u>	<u>9/04/82</u>
COMMERCIAL OPERATION	<u>        </u>	<u>1/1/84</u>

ATTACHMENT B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373  
UNIT: LASALLE ONE  
DATE: March 9, 1984  
COMPLETED BY: DIANE L. LIN  
TELEPHONE: (815) 357-6761

MONTH February 1984

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	926
2.	923
3.	517
4.	161
5.	696
6.	646
7.	685
8.	756
9.	954
10.	1015
11.	966
12.	875
13.	6
14.	0
15.	0
16.	0

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	0
18.	0
19.	0
20.	0
21.	0
22.	0
23.	0
24.	0
25.	0
26.	0
27.	0
28.	0
29.	0
30.	
31.	

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

LTP-300-7  
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ATTACHMENT E  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373  
UNIT NAME LaSalle One  
DATE March 9, 1984  
COMPLETED BY Diane L. Lin  
TELEPHONE (815)357-6761

REPORT MONTH FEBRUARY 1984

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
5	2/3/84	F	21.4	B	3	Rx Scram due to an inadvertent initiation of RCIC during performance of LIS-NB-04.
6	2/13/84	F	407.2	A	3	Rx scram due to "A" condenser boot seal which failed and subsequent loss of condenser vacuum.

## VI. UNIQUE REPORTING REQUIREMENTS

### A. Main Steam Relief Valve Operations for Unit 1.

Relief valve operations during the reporting period are summarized in the following Table. The table included information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

<u>Date</u>	<u>Valves Actuated</u>	<u>No &amp; Type Actuations</u>	<u>Plant Conditions</u>	<u>Description of Events</u>
2/13/84	1B21-F013E	1 Auto	1050 psig	To control pressure after scram
2/13/84	1B21-F013E	1 Manual	1000psig	To control pressure after scram.
2/13/84	1B21-F013C	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013D	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013E	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013V	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013R	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013U	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013S	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013K	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013A	1 Manual	570 psig	LOS-MS-R2

2/13/84	1B21-F013M	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013N	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013G	2 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013H	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013L	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013P	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013F	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013B	1 Manual	570 psig	LOS-MS-R2
2/13/84	1B21-F013J	1 Manual	570 psig	LOS-MS-R2
2/14/84	1B21-F013C	6 Manual	135 psig	LOS-MS-R2
2/14/84	1B21-F013N	5 Manual	135 psig	LOS-MS-R2

#### B. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-159-84	'1B' D/G Electric Motor Driven Air Compressor	To repair air compressor
1-160-84	'1B' D/G	No starting air pressure
1-162-84	'1B' D/G	Recal immersion heater.

#### C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

#### D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.





**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

March 9, 1984

Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station, Unit One, for the period covering February 1 through February 29, 1984.

Very truly yours,

G. J. Diederich  
Station Superintendent  
LaSalle County Station

GJD/DLL/bej

Enclosure

xc: J. G. Keppler, NRC, Region III  
NRC Resident Inspector LaSalle  
Gary Wright Ill. Dept. of Nuclear Safety  
D. P. Galle CEC  
D. L. Farrar CEC  
INPO Records Center  
Ron A. Johnson, PIP Coordinator SNED  
W. R. Jackson, GE Resident

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