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Executive Vice President
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JPN-92-002

U. S. Nuclear Regulatory Commission
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Washington, D.C. 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Proposed Changes to the Technical Specifications
Reactor Vessel Hydrostatic Testing (JPTS-91-014)

REFERENCE: 1. NRC Generic Letter 88-11, dated July 12, 1988, "NRC Position on Radiation Embrittlement of Reactor Vessel Materials and Its Impact on Plant Operations."

Dear Sir:

This application for an amendment to the James A. FitzPatrick Technical Specifications allows performance of the inservice hydrostatic pressure and leak testing of the reactor vessel at temperatures exceeding 212°F.

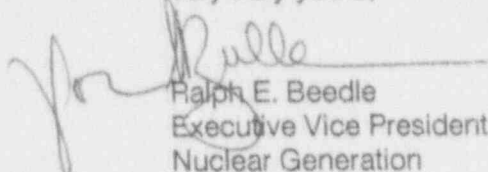
As the accumulated neutron fluence on the reactor vessel increases, the methodology used to calculate hydrostatic testing pressure temperature limits (Reference 1) will ultimately require that these test be conducted above 212°F. The Technical Specifications define hot shutdown mode as the condition when the reactor mode switch is in shutdown and reactor coolant temperature is above 212°F. Applying this definition will require that hydrostatic tests be conducted while the plant is in hot shutdown. However, the current specifications require systems to be operable in the shutdown mode that can not be made operable during the test (e.g., HPCI and RCIC steam lines will be filled with water). The proposed changes will allow hydrostatic testing of the Reactor Coolant System (RCS) above 212°F without requiring the HPCI, RCIC or ADS/SRVs to be operable. All other systems required by technical specification will be maintained operable during the test to protect public health and safety.

The signed original of the Application for Amendment to Operating License is enclosed for filing. Attachments I and II to this application contain the proposed changes to the Technical Specifications and the associated Safety Evaluation.

A copy of this application and the associated attachments are being provided to the designated New York State official in accordance with 10 CFR 50.91.

If you have any questions, please contact J. A. Gray, Jr.

Very truly yours,


Ralph E. Beedle
Executive Vice President
Nuclear Generation

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