Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

March 8, 1984

Mr. H. R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. NUCLEAR REGULATORY COMMISSION
Washington, D.C. 20555

Attention: Mr. J. R. Miller, Chief
Operating Reactors Branch 3

Gentlemen:

DOCKET NOS. 50-266 AND 50-301
ADDITIONAL INFORMATION ON GENERIC LETTER 82-33
SCHEDULE REQUIREMENTS FOR EMERGENCY RESPONSE CAPABILITIES
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In letters dated April 15 and August 24, 1983 Wisconsin Electric provided a description and schedule for implementation of our emergency response activities. Those submittals were in response to Mr. D. G. Eisenhut's letter dated December 17, 1982, Generic Letter 82-33 entitled "Supplement 1 to NUREG-0737 - Requirements for Emergency Response Capability". The August 24 letter updated the schedular commitments we made in the April 15 letter.

We have recently received a request by the vendor of our new plant computer system for an additional one year delay in delivery from June 1984 to June 1985, due primarily to vendor problems in developing the computer system software. The delay affects a number of schedular commitments we have made in the past. This letter details the resultant schedule modifications. The number and letter designation for each item matches those contained in our August 24, 1983 letter.

4. Safety Parameter Display System (SPDS)

As described in our previous submittals, the Safety Assessment System (SAS) is our version of the SPDS and is part of the new computer. Therefore, the SAS installation and operation will similarly be delayed by about one year. We still anticipate that by about six months after delivery, the computer and the SAS can be installed and operational. We therefore anticipate the SAS to be operational on or before December 31, 1985. Also, the description and safety analysis

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submittal scheduled for April 30, 1984 will be delayed until April 30, 1985.

In the August 24, 1983 letter we had noted that certain instruments which are part of the new computer and the SAS, principally the final configuration of the core exit thermocouples, would not be operational until the next outages after computer startup. Even with the additional delay in the computer system delivery, we have continued to proceed with the modification work in containment and with interim installations outside containment such that these instruments can be connected to the computer during computer installation and will not require an additional outage for final connection. They will, therefore, become operational in their final configuration at the same time as the computer system.

5. Detailed Control Room Design Review

Although the SAS will not be operational, we still plan on performing the Control Room Design Review as previously scheduled.

6. Regulatory Guide 1.97-Application to Emergency Response Facilities

We submitted our report describing how we meet the requirements of Type A, B, C, D and E variables as described in Regulatory Guide 1.97 in a letter to the NRC dated September 1, 1983. We are waiting for NRC evaluation of this report. Note that the delay in the computer delivery will delay the completion of the final display configuration of some Regulatory Guide 1.97 instruments until December 1985. However, those affected instruments are or will be in an interim display configuration, in accordance with the schedule in our September 1, 1983 submittal.

7. Upgraded Emergency Operating Procedures (EOP's)

We will be providing further information on the schedule for the new EOP's in a separate letter in the near future.

8. Emergency Response Facilities

8.A.1 Technical Support Center (TSC)

We still plan on completing the emergency power supply to the TSC by December 1984 based on equipment delivery before November 1984. As discussed previously in this letter, operability of the SAS has been delayed until December 1985. Since the final TSC instru-

mentation displays are an integral part of the computer output display system, final operability of the TSC instrumentation is dependent upon the computer and will not be complete until that date. The current plant parameters displayed in the TSC, as described in Table 2.2.2.b-2 of our March 14, 1980 submittal, will continue to be provided using interim loop configurations. (The parameter noted as Pressurizer Wide Range Pressure - Unit 1 and Unit 2 is actually direct Reactor Coolant System Wide Range Pressure - Unit 1 and Unit 2.) A complete description of the plant data collection, storage, analysis and display system will be provided as part of the description and safety analysis discussed in Section 4 of this letter.

8.A.3 Emergency Operations Facility (EOF)

a. Buildings

Since receiving denial from the NRC for our exemption request to locate the Point Beach EOF in our corporate headquarters, we have located the Emergency Support Center in the newly completed Site Boundary Control Center building. Our letter of November 18, 1983 provides a description of the EOF.

b. Instrumentation

EOF instrumentation is dependent on the computer installation schedule discussed earlier.

8.D Meteorology and Dose Assessment

1. Hardware

The primary and backup towers are complete. There is no change in the schedule for the Lake Breeze Effects Tower as discussed in the August 24, 1983 letter.

2. Software

Software development of the meteorology and dose assessment program is essentially complete, except for integration and checkout on the new computer. These latter activities cannot be completed until after delivery of the new computer as discussed earlier. Much of the required cabling is in place, except for a few

short runs and the computer terminations themselves which are dependent on installation of the new computer.

3. Submittal

As stated in the August 24 letter, we still expect to provide a complete description of the meteorological and dose assessment hardware and software by October 1, 1984.

Other items not specifically addressed in the April 15 and August 24 letters, which will not be in their final configuration until the computer is completed, include the subcooling monitors and the Reactor Vessel Water Level System. Currently one of two subcooling monitor channels from each unit is connected to a subcooling display temporarily located in an auxiliary rack in the computer room. The second channel from each unit is connected to the backup computer with continuous CRT display in the control room. These subcooling displays are based on wide range RCS pressures and wide range hot leg loop (RTD) temperatures. The final configuration will include the capability to display subcooling based on thermocouple temperatures as well. Currently a subcooling indication based on the thermocouples is available through the existing plant process computers. One uncompensated reactor vessel water level channel from each unit is connected to the backup computer with continuous CRT display in the control room. Compensated level will be available when the new computer becomes operational.

As stated earlier in this letter, we have received a formal request from our computer system vendor to revise the contract schedule for delivery of the system from June 1984 to June 1985. We have not yet indicated our concurrence with the vendor's requested schedule revision, pending resolution of other related contractual matters and the provision of reasonable assurance that a new schedule will be met. We will keep you advised of any schedular developments, and we will continue to do all we can to complete this project in an acceptable manner and on a timely basis. Should you wish to discuss these matters further, please contact us.

Very truly yours,

C. W. Fay

Vice President-Nuclear Power

C. W. Fay

Copy to NRC Resident Inspector