Attachment 1

Proposed Administrative Technical Specification Changes

1) Change to Note 3 of Table 4.3-1 (Both Units 1 & 2)

The existing note requires that the monthly time clock for the comparison of incore to excore axial flux differences begin when the reactor is above 15% RATED THERMAL POWER. This requirement is not consistent with the surveillance requirements of Technical Specification sections 4.2.2.2.d.1(b) and 4.2.3.1.b which require that flux mapping be performed every 31 EFPD. Since flux mapping is required in order to verify incore to excore axial flux differences, it is requested that all these Technical Specification requirements be performed every 31 EFPD.

This change will alleviate the administrative burden of maintaining separate schedules and the potential for duplicate flux mapping solely for incore to excore axial flux difference comparisions. Reduction of this duplication of operator effort will allow more time and attention for more significant issues of plant operation and reduce wear on the movable incore detector system components.

2) a) Change to Table 3.6-1 Section A, Items 42, 43, 44 and 45 (Both Units 1 and 2)

Recent plant modifications to implement license commitments related to containment purge have been completed. As part of these modifications the Containment Mini-purge Isolation Valves have been replaced with 8 inch diameter valves. The valve identification numbers have been changed as a result. This renumbering requires these Technical Specification Tables to be revised.

b) Change to Table 3.6-1, Section D, Items 1,2,5,6 and 17 (Unit 1 only)

This change simply corrects the valve numbers so that the proper Unit 1 prefix number is reflected in the tables.

3) Change to Figure 6.2-2 (Both Units 1 & 2)

The Chemistry Supervisor Position title has been changed to Chemistry and Environmental Supervisor. There have not been significant responsibility changes as a result of this change. The change in title simply reflects more closely the responsibilities of this position.

In addition, the new position of Computer Services Supervisor has been added to Figure 6.2-2. This change reflects the importance of the Computer Services function in supporting the operation of the Farley Nuclear Plant.

4) Change to Technical Specification 3.7.11.1.c (Unit 1 only)

A recent review of this Technical Specification section identified that the cross-references to other fire protection specifications were in error. The proper cross-references are to Sections 3.7.11.2, 3.7.11.4 and 3.7.11.5. This corrects an administrative error that has existed since License Amendment 26 was issued.

8403130463 840301
PDR ADOCK 05000348
P PDR

Attachment 2

Proposed Changes to
Technical Specification Pages

Unit 1

Page 3/4 3-14
Page 3/4 6-17
Page 3/4 6-18
Page 6-3
Page 3/4 7-82

Unit 2

Page 3/4 3-14
Page 3/4 6-17
Page 6-3

TABLE 4.3-1 (Continued)

TABLE NOTATION

* - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.

(1) - If not performed in previous 7 days.

(2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.

(3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.

(4) - Manual ESF functional input check every 18 months.

(5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.

(6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.

(7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.

(8) - Logic only, if not performed in previous 92 days.

(9) - CHANNEL FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip after latching the turbine.

(10) - If not performed in the previous 31 days.

TABLE 3.6-1 (Continued)

	FUNCTION	ISOLATION TIME (SEC)
PHASE "A" ISOLATION (Continued		
32. CCW-HV-3067	CCW from exc. letdown RCDT HXS	<10
33. CVC-HV-8860	Accumulators fill line isolation	<10
34. SS-HV-3766	Accumulator tanks sample isolation valve	<10
35. SS-HV-3334	Accumulator tanks sample isolation valve	<10
36. LWP-HV-7126	RCDT vent line isolation valve	<10
37. LWP-HV-7150	RCDT vent line isolation valve	<10
38. LWP-HV-3380	Containment sump recirculation valve	<10
39. CTS-HV-3659	Demineralizer water to reactor MD storage	<10
40. CBV-HV-3196	Containment purge exhaust isolation valve	<5
41. CBV-HV-3197	Containment purge supply isolation valve	<5
42. CBV-HV-2867 C	Containment mini-purge exhaust isolation valve	<5
43. CBV-HV-2867 D	Containment mini-purge exhaust isolation valve	<5
44. CBV-HV-2866 C	Containment mini-purge supply isolation valve	<5
45. CBV-HV-2866 D	Containment mini-purge supply isolation valve	<5
B. PHASE "B" ISOLATION		
1. CCW-MOV-3052	CCW to RCP coolers	<15
2. CCW-MOV-3046	CCW from RCP oil coolers	<15
3. CCW-MOV-3182	CCW from RCP oil coolers	<15
4. CCW-HV-3184	CCW from RCP THRM BARR	<10
5. CCW-HV-3045	CCW from RCP THRM BARR	<10
6. IA-HV-3611	Containment instrument air supply valve	<10
C. SAFETY INJECTION SIGNAL		
1. CVC-MOV-8107	Charging pumps to regenerative HX	<10
2. CVC-MOV-8108	Charging pumps to regenerative HX	<10
3. SW-MOV-3135	SW to RCP motor air coolers	<15
4. SW-MOV-3131	SW from RCP motor air coolers	<15
5. SW-MOV-3134	SW from RCP motor air coolers	<15

TABLE 3.6-1 (Continued)

		FUNCTION	ISOLATION TIME (SEC)
9.	MANUAL		
1.	Q1G31V012	Refueling Cavity Supply	NA
2.	Q1G21V005*	Reactor Coolant drain tank	NA
3.	RHR-MOV-8701A**	Reactor coolant LP IC to RHR pump 1A	<120
4.	RHR-MOV-8702A**	Reactor coolant LP 1A to RHR pump 1B	<120
5.	Q1P18V001*	Service Air	NA
6.	Q1P18V002*	Service Air	NA
7.	CBV-MOV-3238	Containment leak rate test valve	NA
8.	CBV-MOV-3239	Containment leak rate test valve	NA
9.	RHR-MOV-8811 A	Containment sump to RHR pump 1A	<17
10.	RHR-MOV-8811 B	Containment sump to RHR pump 1B	<17
11.	RHR-MOV-8812 A	Containment sump to RHR pump 1A	<17
12.	RHR-MOV-8812 B	Containment sump to RHR pump 1B	<17
13.	CS-MOV-8826 A	CS pump 1A containment sump suction isolation	<17
14.	CS-MOV-8826 B	CS pump 1B containment sump suction isolation	<17
15.	CS-MOV-8827 A	CS pump 1A containment sump suction isolation	<17
16.	CS-MOV-8827 B	CS pump 1B containment sump suction isolation	<17
17.	Q1B13V026B*	Pressurizer pressure generator	NA
18.	CBV-MOV-3528 A*	Containment post-LOCA sampling valve 1	NA
19.	CBV-MOV-3528 B*	Containment post-LOCA sampling valve 2	NA
20.	CBV-MOV-3528 C*	Containment post-LOCA sampling valve 3	NA
21.	CBV-MOV-3528 D*	Containment post-LOCA sampling valve 4	NA
22.	CBV-MOV-3739 A*	Containment post-LOCA sampling isolation valve	NA
23.	CBV-MOV-3739 B*	Containment post-LOCA sampling isolation valve	NA
24.	CBV-MOV-3745 A*	Containment post-LOCA sampling return valve	NA
25.	CBV-MOV-3745 B*	Containment post-LOCA sampling return valve	NA
26.	CBV-MOV-3835 A*	Containment post-LOCA sampling return valve	NA
27.	CBV-MOV-3835 B*	Containment post-LOCA sampling return valve	NA
28.	CBV-MOV-3740*	Containment post-LOCA vent isolation valve	NA
29.	CBV-MOV-3530*	Containment post-LOCA vent isolation valve	NA

*May be opened on an intermittent basis under administrative controls

**May be opened and power removed under administrative controls when the plant is in MODE 4 (for ensuring overpressure protection system operability).

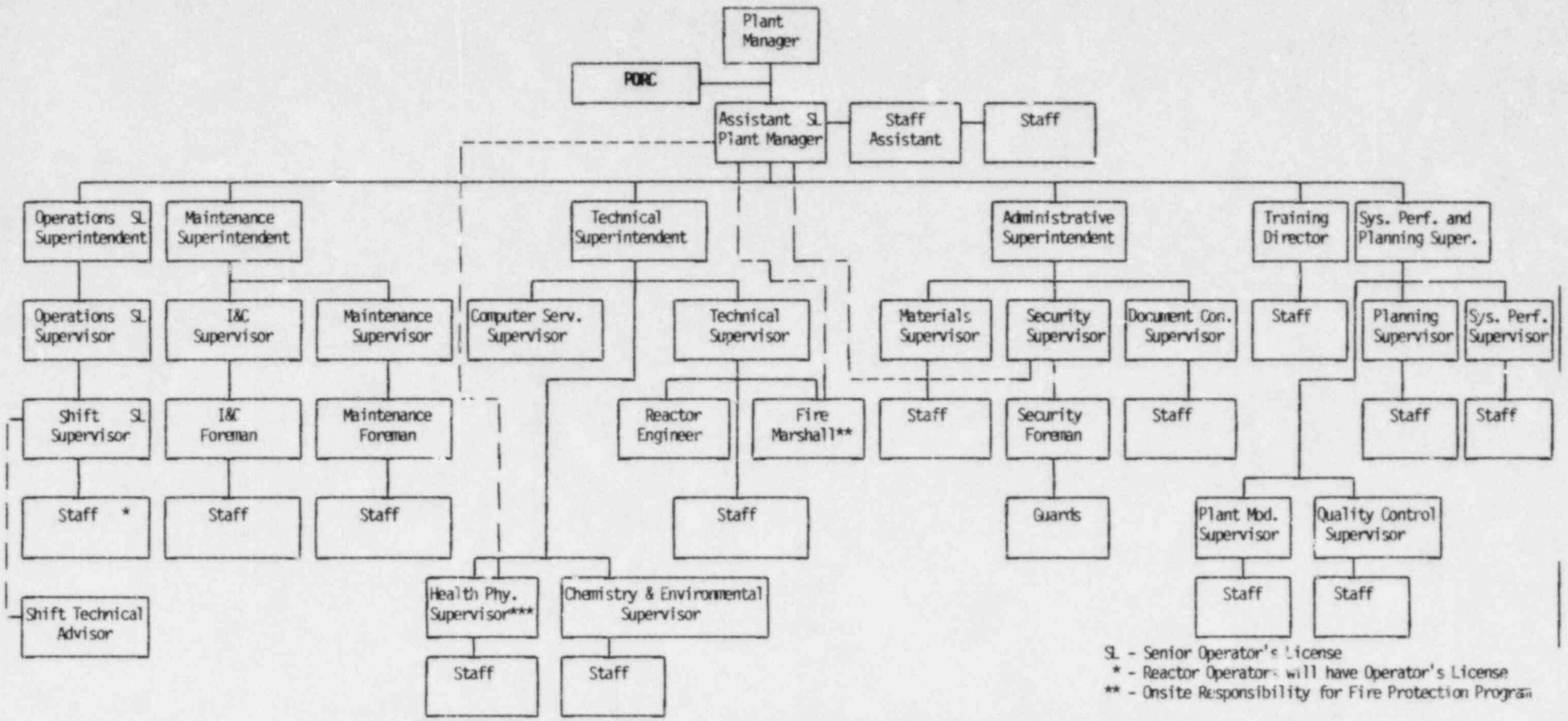

Figure 6.2-2 Facility Organization

*** In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, in matter of radiation policy determination, interpretation or implementation (Based upon the Health Physic Supervisor's judgment) the Health Physics Supervisor may report directly to the Assistant Plant Manager.

PLANT SYSTEMS

3/4.7.11 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.11.1 The fire suppression water system shall be OPERABLE with:

a. Two high pressure pumps, each with a capacity of 2500 gpm, with their discharge aligned to the fire suppression header,

b. Separate water supplies, each with a minimum contained volume of 250,000 gallons, and

c. An OPERABLE flow path capable of taking suction from each tank and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves, the last valve ahead of the water flow alarm device on each sprinkler or hose standpipe, and the last valve ahead of the deluge valve on each deluge or spray system required to be OPERABLE per Specification 3.7.11.2, 3.7.11.4 and 3.7.11.5.

APPLICABILITY: At all times.

ACTION:

a. With one of the above required pumps and/or water supplies inoperable, restore the inoperable equipment to OPERABLE status within 7 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

b. With the fire suppression water system otherwise inoperable:

1. Establish a backup fire suppression water system within 24 hours, and

2. In lieu of any other report required by Specification 6.9.1, submit a Special Report in accordance with Specification 6.9.2:

a) By telephone within 24 hours,

b) Confirmed by telegraph, mailgram or facsimile transmission no later than the first working day following the event, and

c) In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

TABLE 4.3-1 (Continued)

TABLE NOTATION

* - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.

(1) - If not performed in previous 7 days.

(2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.

(3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.

(4) - Manual ESF functional input check every 18 months.

(5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.

(6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.

(7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.

(8) - Logic only, if not performed in previous 92 days.

(9) - CHANNEL FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip after latching the turbine.

(10) - If not performed in the previous 31 days.

TABLE 3.6-1 (Continued)

	FUNCTION	ISOLATION TIME (SEC)
PHASE "A" ISOLATION (Continued		
32. CCW-HV-3067	CCW from exc. letdown RCDT HXS	<10
33. CVC-HV-8860	Accumulators fill line isolation	<10
34. SS-HV-3766	Accumulator tanks sample isolation valve	<10
35. SS-HV-3334	Accumulator tanks sample isolation valve	<10
36. LWP-HV-7126	RCDT vent line isolation valve	<10
37. LWP-HV-7150	RCDT vent line isolation valve	<10
38. LWP-HV-3380	Containment sump recirculation valve	<10
39. CTS-HV-3659	Demineralizer water to reactor HD storage	<10
40. CBV-HV-3196	Containment purge exhaust isolation valve	<5
41. CBV-HV-3197	Containment purge supply isolation valve	<5
42. CBV-HV-2867 C	Containment mini-purge exhaust isolation valve	<5
43. CBV-HV-2867 D	Containment mini-purge exhaust isolation valve	<5
44. CBV-HV-2866 C	Containment mini-purge supply isolation valve	<5
45. CBV-HV-2866 D	Containment mini-purge supply isolation valve	<5
B. PHASE "B" ISOLATION		
1. CCW-MOV-3052	CCW to RCP coolers	<15
2. CCW-MOV-3046	CCW from RCP oil coolers	<15
3. CCW-MOV-3182	CCW from RCP oil coolers	<15
4. CCW-HV-3184	CCW from RCP THRM BARR	<10
5. CCW-HV-3045	CCW from RCP THRM BARR	<10
6. IA-HV-3611	Containment instrument air supply valve	<10
C. SAFETY INJECTION SIGNAL		
1. CVC-MOV-8107	Charging pumps to regenerative HX	<10
2. CVC-MOV-8108	Charging pumps to regenerative HX	<10
3. SW-MOV-3135	SW to RCP motor air coolers	<15
4. SW-MOV-3131	SW from RCP motor air coolers	<15
5. SW-MOV-3134	SW from RCP motor air coolers	<15

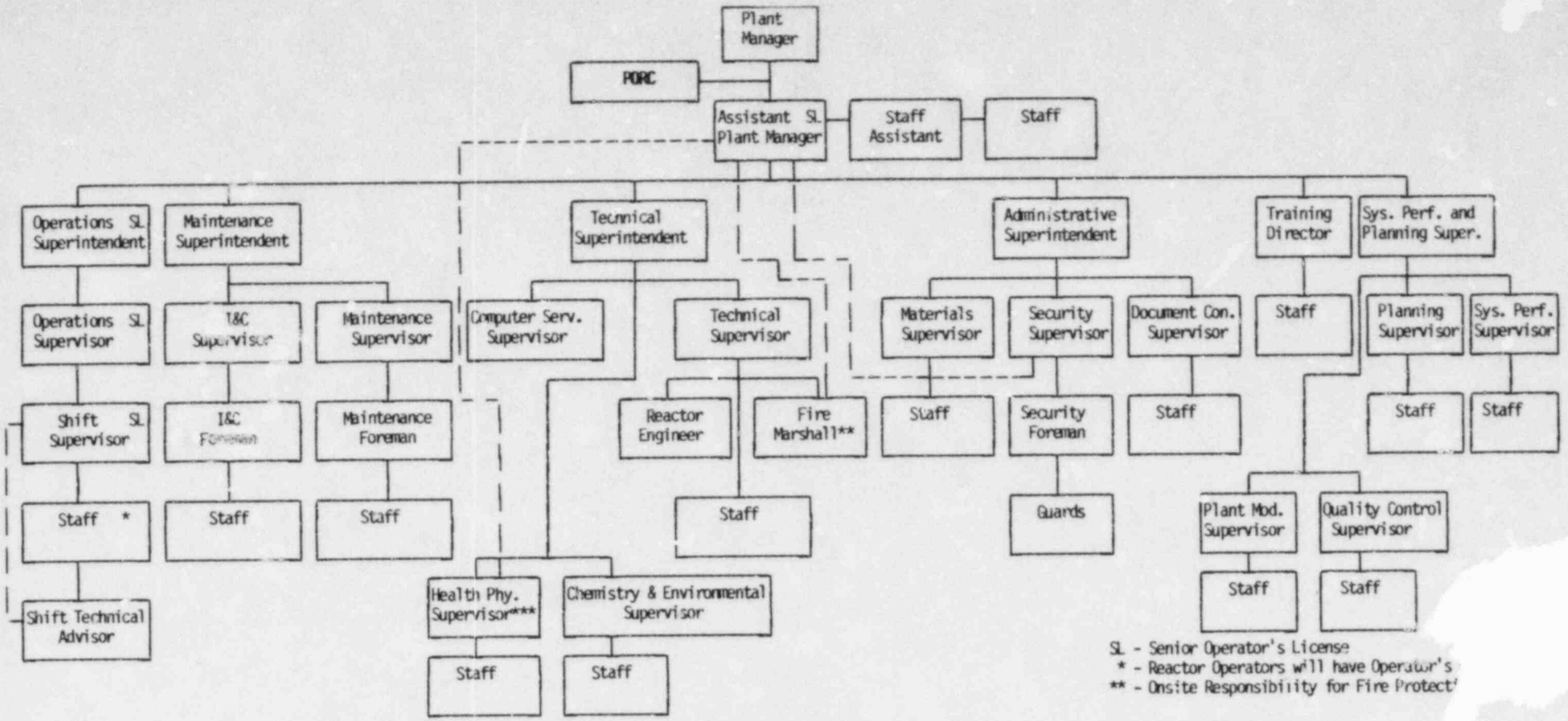

Figure 6.2-2 Facility Organization

*** In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, in matter of radiation policy determination, interpretation or implementation (Based upon the Health Physics Supervisor's judgment) the Health Physics Supervisor may report directly to the Assistant Plant Manager.