

LICENSEE EVENT REPORT (LER)

APPROVED CMB NO. 2180-010N
EXPIRES - 6/31/85

FACILITY NAME (1)										DOCKET NUMBER (2)										PAGE (2)																													
INDIAN POINT UNIT 2										0 5 0 0 0 2 4 7										1 OF 0 2																													
TITLE (4)																																																	
MSIV EXCESSIVE CLOSURE TIME																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER (8)												
0 1			0 5			8 4			8 4			- 0 0 2			- 0 0			0 2			0 4			8 4													0 5 0 0 0 1 1												
OPERATING MODE (9)										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																							
POWER LEVEL (10) 0 0 1 0										20.402(a)										20.405(a)										50.73(a)(2)(iv)										73.71(a)									
										20.405(a)(1)(i)										50.36(a)(1)										50.73(a)(2)(v)										73.71(a)									
										20.405(a)(1)(ii)										50.36(a)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 305A)									
										20.405(a)(1)(iii)										50.73(a)(2)(ii)										50.73(a)(2)(vii)(A)																			
										20.405(a)(1)(iv)										50.73(a)(2)(iii)										50.73(a)(2)(viii)(B)																			
20.405(a)(1)(v)										50.73(a)(2)(iv)										50.73(a)(2)(ix)(C)																													
20.405(a)(1)(vi)										50.73(a)(2)(v)										50.73(a)(2)(x)(D)																													
20.405(a)(1)(vii)										50.73(a)(2)(vi)										50.73(a)(2)(xi)(E)																													
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME																				TELEPHONE NUMBER																													
MICHAEL BLATT, DIRECTOR-REGULATORY AFFAIRS																																																	
AREA CODE																																																	
9 1 1 4										5 2 6 5 1 1 2 7 1																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																						
B			S B I I S I V			A I 5 1 8 1 5			Y																																								
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If yes, complete EXPECTED SUBMISSION DATE)																				NO										EXPECTED SUBMISSION DATE (15)																			
ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-spaced typewritten lines) (16)																																																	
<p>On January 5, 1984 while the plant was shutdown for Maintenance, two of the four Main Steam Isolation Valves (MSIV) did not fully close within the five (5) second Technical Specification requirement. The valves' shaft lubrication had dried out, thereby increasing the friction on the valve shaft. The valve shaft and packing were lubricated. The valves were then satisfactorily tested by closing within five (5) seconds.</p>																																																	
8403130405 840204 PDR ADDOCK 05000247 S PDR																																																	

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/95

FACILITY NAME (1) INDIAN POINT, UNIT 2	DOCKET NUMBER (2) 0500024784-002-00	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
					2	OF 2

TEXT (if more space is required, use additional NRC Form 365a (17))

On January 5, 1984, at 2220 hours the four Main Steam Isolation Valves (MS-1) were manually signaled to close after the main steam flow had stopped. MS-1-21 and MS-1-22 did not close within the five (5) second time limit specified in Technical Specification 3.4A(5). MS-1-22 needed approximately 20 seconds to fully close and MS-1-21 required manual assistance to fully close.

The MS-1 isolation valves contain free swinging discs which are held in the open position out of the main steam flow path by spring loaded air pistons. The valves will close automatically upon receipt of a signal from the overall unit protection system and are installed so that the disk will drop into the main steam flow path and will be forced closed by the main steam flow.

The inability of the MSIVs to fully close or close within the specified time period with no steam flow is attributed to the friction caused by loss of shaft lubricant. The lubricant is believed to have vaporized because of the high temperature environment. During periods of operation only the packing can be lubricated not the shaft.

The valve shafts and packing of both valves were lubricated with FELPRO-5000. Post maintenance testing verified the valves closed within five (5) seconds. An engineering review has been started to evaluate the current packing design, lubrication practice, and possible modification of the valves, to assure adequate shaft lubrication when the unit is on-line. During the upcoming refueling outage, preventive maintenance and inspection will be performed on the valve shaft.

The MSIVs would have been able to perform their design safety function since both valve discs were in the steam path and would have been assisted to close had there been main steam flow.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

February 27, 1984

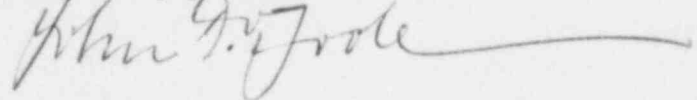
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-84-002-00

Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Dear Dr. Murley:

The attached Licensee Event Report LER-84-002-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Thomas Foley, Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

IE 22
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