

ATTACHMENT 1

Unit 1

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Revision

Replace

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Revision

Replace

ADMINISTRATIVE CONTROLS

- e. Type of container (e.g., LSA, Type A, Type B, Large Quantity) and
- f. Solidification agent (e.g., cement, urea formaldehyde).

The radioactive effluent release reports shall include unplanned releases from the site to unrestricted areas of radioactive materials in gaseous and liquid effluents on a quarterly basis.

The radioactive effluent release reports shall include any changes to the PROCESS CONTROL PROGRAM (PCP) made during the reporting period.

MONTHLY OPERATING REPORT

6.9.1.10 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORV's or safety valves, shall be submitted on a monthly basis to the Commission, pursuant to 10 CFR 50.4, no later than the 15th of each month following the calendar month covered by the report.

Any changes to the OFFSITE DOSE CALCULATION MANUAL shall be submitted with the Monthly Operating Report within 90 days in which the change(s) was made effective. In addition, a report of any major changes to the radioactive waste treatment systems shall be submitted with the Monthly Operating Report for the period in which the change was implemented.

RADIAL PEAKING FACTOR LIMIT REPORT

6.9.1.11 The F_{xy} limit for Rated Thermal Power (F_{xy}^{RTP}) for all core planes containing bank "D" control rods and all unrodded core planes shall be established and documented in the Radial Peaking Factor Limit Report before each reload cycle (prior to MODE 2) and provided to the Commission, pursuant to 10 CFR 50.4, upon issuance. In the event that the limit would be submitted at some other time during core life, it will be submitted upon issuance, unless otherwise exempted by the Commission.

Any information needed to support F_{xy}^{RTP} will be by request from the NRC and need not be included in this report.

ANNUAL DIESEL GENERATOR RELIABILITY DATA REPORT

6.9.1.12 The number of tests (valid or invalid) and the number of failures to start on demand for each diesel generator shall be submitted to the NRC annually. This report shall contain the information identified in Regulatory Position C.3.b of NRC Regulatory Guide 1.108, Revision 1, 1977.

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ATTACHMENT 2

Evaluation of No Significant Hazards Consideration

SIGNIFICANT HAZARDS CONSIDERATION ANALYSIS

Administrative Change for Radial Peaking Factor Limit Reporting

The current reporting requirement for radial peaking factor limit reporting in Section 6.9.1.11 of the Farley Technical Specifications is 60 days prior to criticality. To allow for more flexibility after shutdown, in particular for emergency core redesigns, this reporting requirement has been revised to require submittal before each reload cycle. The new requirement is consistent with reporting requirements for Core Operating Limits Reports (COLRs). There are no safety analyses or design criteria that are sensitive to this administrative change, and the same reporting requirements will be continued to be met for the reporting time limit change.

Based on the information presented above, the following conclusions can be reached with respect to 10 CFR 50.92:

1. Since the reporting requirements are an administrative change, this change does not involve a significant increase in the probability or consequences of an accident previously evaluated in the Farley FSAR. With the exception of the reporting schedule, no other changes to the reporting requirements are made.
2. This administrative change in reporting requirements does not create the possibility of a new or different kind of accident from any previously evaluated in the Farley FSAR since only the time for reporting requirements has changed. Therefore, the probability of a new or different kind of accident is not created.
3. This administrative change is not safety-related and therefore does not involve any reduction in the margin of safety.

Based on the preceding information, it has been determined that the proposed change in reporting times for the radial peaking factors does not involve a significant hazards consideration as defined in 10 CFR 50.92 (c).