

LICENSEE EVENT REPORT (LER)

50-389

FACILITY NAME (1) St. Lucie Unit 2										DOCKET NUMBER (2) 0 5 0 0 0										PAGES 1 OF 1																															
TITLE (4) Reactor Trip Due to Low Steam Generator Level																																																			
EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)																																										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES NA			DOCKET NUMBER (5) 0 5 0 0 0																																							
0	2	0	9	8	4	8	4	0	0	4	0	0	0	3	0	9	8	4	0	5	0	0	0																												
OPERATING MODE (3)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																																	
1		20.402(b)										20.408(a)										X 80.73(a)(2)(iv)										73.71(b)																			
POWER LEVEL (10)		1 1 0 0										20.408(a)(1)(i)										80.36(a)(1)										80.73(a)(2)(v)										73.71(a)									
		20.408(a)(1)(ii)										80.36(a)(2)										80.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 308A)																			
		20.408(a)(1)(iii)										80.73(a)(2)(i)										80.73(a)(3)(vi)(A)																													
		20.408(a)(1)(iv)										80.73(a)(2)(ii)										80.73(a)(3)(vi)(B)																													
		20.408(a)(1)(v)										80.73(a)(2)(iii)										80.73(a)(2)(vii)(B)																													
		20.408(a)(1)(vi)										80.73(a)(2)(iv)										80.73(a)(2)(x)																													
LICENSEE CONTACT FOR THIS LER (12)																																																			
NAME Bruce Parks, Senior Plant Engineer												TELEPHONE NUMBER AREA CODE 3 0 5 4 6 5 - 3 5 5 0																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																																									
B	S J	P	I 0 7 5	Y		B	S B R V	C 7 1 0	Y																																										
X	S D	P	A 1 8 0	N																																															
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR																																			
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO																																							
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																			
<p>While at 100% power the main feedwater pump tripped due to low suction pressure. The reactor subsequently tripped on Low SG Level. Following the trip the 2C auxiliary feedwater pump started then tripped on overspeed. A single steam safety on the "A" steam generator also stuck partially open following the trip. Approximately 40 minutes was required to reseal the safety. Cooldown from the open safety was successfully controlled.</p> <p>The cause of the low suction pressure to the feedwater pumps was not positively determined. Condensate pump vent line design probably contributed. The design is being changed to allow proper venting during strainer cleaning.</p> <p>The 2C auxiliary feedwater pump tripped due to transients on a power supply during auxiliary feedwater actuation. The cause was not immediately known. However, repeated testing, over a period of 6 days, revealed the problem. A modification has been completed to eliminate this problem.</p> <p>The safety stuck open because a cotter pin was missing from a spindle nut. When the safety opened (as expected) the nut vibrated down and held the valve partially open. The nut pin was replaced and all safeties were checked.</p> <p>This is the first LER of a reactor trip caused by a trip of a feedwater pump.</p>																																																			
8403130035 840309 PDR ADDOCK 05000389 S PDR																																																			



March 9, 1984
PNS-LI-14-86

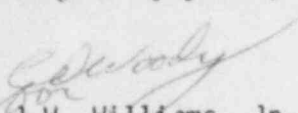
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 84-04
St. Lucie Unit 2
Date of Event: February 9, 1984
Reactor Trip Due to Low Steam Generator Level

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,


J.W. Williams, Jr.
Vice President
Nuclear Energy

JWW/PLP/js

cc: J.P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
File 933.1

IE22
1/1