

LICENSEE EVENT REPORT

A0-S2-75-12

CONTROL BLOCK

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME 01 V A S P S 2										LICENSE NUMBER 00-000000-00										LICENSE TYPE 41110					EVENT TYPE 01							
7	8	9	14	15	25	26	30	31	32	7	8	9	14	15	25	26	30	31	32	7	8	9	14	15	25	26	30	31	32			
CATEGORY 01 CONT			REPORT TYPE P 0		REPORT SOURCE T L		DOCKET NUMBER 050-0281					EVENT DATE 070675					REPORT DATE 071475															
7	8	9	57	58	59	60	61	68	69	74	75	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100

EVENT DESCRIPTION

02 During full power operation, the main steam trip valve in B steam line suddenly closed

03 causing the closure of A and C trip valves and a reactor trip. While at hot shutdown

04 testing revealed that the A and C main steam trip valves would not close following

05 reopening. This event is considered an abnormal occurrence since the failure of the

06 trip valves to close during testing is an engineered safeguard system malfunction (cont)

SYSTEM CODE H B		CAUSE CODE E	COMPONENT CODE V A L V E X				PRIME COMPONENT SUPPLIER A	COMPONENT MANUFACTURER S O 7 5			VIOATION N																						
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40

CAUSE DESCRIPTION

08 Investigation revealed a faulty piston O-ring in one of the operating cylinders on B

09 trip valve and trash in the solenoid valve supplying air to the cylinder. Adequate

10 air pressure was not maintained due to blockage in the solenoid valve and leakage (cont)

FACILITY STATUS G		% POWER 000		OTHER STATUS N/A		METHOD OF DISCOVERY C		DISCOVERY DESCRIPTION N/A																									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40
FORM OF ACTIVITY RELEASED Z		CONTENT OF RELEASE Z		AMOUNT OF ACTIVITY N/A		LOCATION OF RELEASE N/A																											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40

PERSONNEL EXPOSURES

NUMBER 000	TYPE Z	DESCRIPTION N/A																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40

PERSONNEL INJURIES

NUMBER 000	DESCRIPTION N/A																																
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40

OFFSITE CONSEQUENCES

15 N/A

LOSS OR DAMAGE TO FACILITY

TYPE Z	DESCRIPTION N/A																																
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40

PUBLICITY

17 N/A

ADDITIONAL FACTORS

18 N/A

19

8403120366 760120
PDR ADOCK 05000281
S PDR

NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

EVENT DESCRIPTION: (con't)

which could render the system incapable of performing its intended function (TS 1.1.6). AO-S2-75-12

CAUSE DESCRIPTION: (con't)

past the O-ring causing the valve to trip. The O-ring was replaced and the air lines cleared of all debris. A and C trip valves would not close due to binding between the back of the valve discs and valve body. Non-destructive testing revealed no component failures on any valves. The chamfer corner on the edge of the valve discs was rounded by grinding in the area where binding occurred. The corners were ground sufficiently on all valves to allow the disc to be opened approximately two degrees higher than the movement when connected to the operating cylinders. The load carrying capacity of the discs is not changed by this modification.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

July 15, 1975

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 598
PO&M/JTB:clw

Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.1, the Virginia Electric and Power Company hereby submits forty (40) copies of Abnormal Occurrence Report No. AO-S2-75-12.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosures

40 copies of AO-S2-75-12

cc: Mr. K. R. Goller

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