

# LICENSEE EVENT REPORT

A0-S1-75-14

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 01 V A S P S 1 14  
 LICENSE NUMBER: 00-000000-000 25  
 LICENSE TYPE: 41110 30  
 EVENT TYPE: 01 32

CATEGORY: 01 CON'T 57  
 REPORT TYPE: P 58  
 REPORT SOURCE: T 59  
 DOCKET NUMBER: 050-0280 68  
 EVENT DATE: 080175 74  
 REPORT DATE: 081275 80

## EVENT DESCRIPTION

02 During startup operations on Unit No. 1, sampling of the Safety Injection Accumulators  
 03 revealed that the boron concentration in 1C Accumulator was approximately 1850 ppm.  
 04 The reactor was critical in a hot standby status at the time the deviation was de-  
 05 tected. This event is considered an abnormal occurrence since Technical Specifica-  
 06 tion 3.3.A.2 requires that the boron concentration be at least 1950 ppm (con't)

SYSTEM CODE: 07 S F 10  
 CAUSE CODE: A 11  
 COMPONENT CODE: A C C U M U 17  
 PRIME COMPONENT SUPPLIER: Z 43  
 COMPONENT MANUFACTURER: Z 9 9 9 47  
 VIOLATION: Y 48

## CAUSE DESCRIPTION

08 The 1C Accumulator was diluted during unit heat-up operations on the previous day.  
 09 Inleakage, evidenced by an increase in accumulator level, occurred through the loop  
 10 check valves when the accumulator discharge valve was opened at a primary (con't)

FACILITY STATUS: 11 C 9  
 % POWER: 004 12  
 OTHER STATUS: N/A 13  
 METHOD OF DISCOVERY: B 44  
 DISCOVERY DESCRIPTION: N/A 46  
 FORM OF ACTIVITY RELEASED: Z 9  
 CONTENT OF RELEASE: 2 10  
 AMOUNT OF ACTIVITY: N/A 11  
 LOCATION OF RELEASE: N/A 45

## PERSONNEL EXPOSURES

13 NUMBER: 000 11  
 TYPE: Z 12  
 DESCRIPTION: N/A 13

## PERSONNEL INJURIES

14 NUMBER: 000 11  
 DESCRIPTION: N/A 12

## OFFSITE CONSEQUENCES

15 N/A

## LOSS OR DAMAGE TO FACILITY

16 TYPE: 1  
 DESCRIPTION: N/A 10

## PUBLICITY

17 N/A

## ADDITIONAL FACTORS

18 N/A

19 N/A

8403120328 750812  
 PDR ADOCK 05000280  
 S PDR

NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

EVENT DESCRIPTION: (continued)

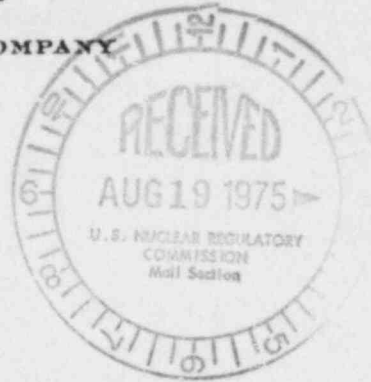
whenever the reactor is critical. The reactor was made subcritical and the boron concentration increased to the correct concentration. The reactor was critical for approximately 8 hours while the accumulator boron concentration was below the specified limit. (A0-S1-75-14)

CAUSE DESCRIPTION: (continued)

system pressure of approximately 950 psig. In order to terminate the inleakage, primary system pressure was increased sufficiently to seat the check valves. The accumulator was then drained to the proper level. In order to prevent recurrence, the operating procedures will be revised to require that the accumulators be sampled after the discharge valves are opened during the heat-up operations and the concentration verified to be within specification prior to criticality.

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

August 12, 1975



Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 646  
PO&M/JTB:nkw

Docket No. 50-280  
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.1, the Virginia Electric and Power Company hereby submits forty (40) copies of Abnormal Occurrence Report No. AO-S1-75-14.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosures  
40 copies of AO-S1-75-14

cc: Mr. K. R. Goller

50-280  
*signing*

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