

# LICENSEE EVENT REPORT

A0-S1-75-17

CONTROL BLOCK

(PLEASE PRINT ALL REQUIRED INFORMATION)

6

LICENSEE NAME 01 V A S P S I										LICENSE NUMBER 00-000000-00										LICENSE TYPE 41110					EVENT TYPE 01		
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32		
01 CONT		CATEGORY M I		REPORT TYPE T		REPORT SOURCE L		DOCKET NUMBER 050-0280										EVENT DATE 091575					REPORT DATE 092375				
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32		

## EVENT DESCRIPTION

02 During normal operation, it was found that one channel of the redundant heat tracing																															
03 required by Technical Specification 3.2 had failed. The redundant channel was																															
04 operable. A0-S1-75-17																															
05																															
06																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

SYSTEM CODE 07 W A				CAUSE CODE 08 E		COMPONENT CODE 09 P 1 P E X X						PRIME COMPONENT SUPPLIER 10 A		COMPONENT MANUFACTURER 11 A 3 5 0				VIOLATION 12 Y							
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32

## CAUSE DESCRIPTION

08 An investigation revealed that the breaker for heat tracing panel #1 had tripped on																															
09 overload due to its associated power transformer being short circuited by a service																															
10 water leak. The leak originated from a failure in temporary service water (con't)																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

FACILITY STATUS 11 E		% POWER 12 1 0 0		OTHER STATUS 13 N/A								METHOD OF DISCOVERY 14 A		DISCOVERY DESCRIPTION 15 N/A											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32

FORM OF ACTIVITY RELEASED 12 Z		CONTENT OF RELEASE 13 Z		AMOUNT OF ACTIVITY 14 N/A										LOCATION OF RELEASE 15 N/A											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32

## PERSONNEL EXPOSURES

NUMBER 13 0 0 0		TYPE 14 Z		DESCRIPTION 15 N/A																											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

## PERSONNEL INJURIES

NUMBER 14 0 0 0		DESCRIPTION 15 N/A																													
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

## OFFSITE CONSEQUENCES

15 N/A																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

## LOSS OR DAMAGE TO FACILITY

TYPE 16 Z		DESCRIPTION 17 N/A																													
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

## PUBLICITY

17 N/A																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

8403120307 750923  
PDR ADOCK 05000280  
S PDR

## ADDITIONAL FACTORS

18 The health and safety of the public was in no way affected by this event.																															
19																															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						

NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

CAUSE DESCRIPTION (cont'd)

piping which was associated with an earlier service water piping leak. The leak was secured and the transformer was repaired. The heat tracing channel was tested and returned to service approximately eight hours after the failure. The piping was manufactured by Ameron.

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

September 25, 1975



Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 711  
PO&M/JTB:clw

Docket Nos. 50-280  
License Nos. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.1,  
the Virginia Electric and Power Company hereby submits forty (40) copies  
of Abnormal Occurrence Report No. AO-S1-75-17.

The substance of this report has been reviewed by the Station Nuclear  
Safety and Operating Committee and will be placed on the agenda for the  
next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosures

40 copies of AO-S1-75-17

cc: Mr. K. R. Goller

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